

Sources of energetic particles in fusion Plasmas

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Sources of energetic ions

- Nuclear reactions, especially alpha particles generated in D-T reactions
 - Good confinement of alpha particles is obviously essential for a fusion plasma
 - Fast ion instabilities driven by is a concern for ITER alpha particle heating
- Neutral Beam Injection (NBI) for auxiliary plasma heating
 - The work horse for plasma heating in most present day devices
 - Mostly ion heating in current day devices $E_{inj} \sim 100$ keV;
 - Less ion heating in ITER $E_{inj} \sim 1$ MeV, especially in ramp-up phase
 - Can induce strong plasma rotation and non-inductive currents
- Acceleration of ions by Radio Frequency (RF) for auxiliary heating
 - Complicated wave physics + wave particle interaction, relies on velocity space diffusion for absorption; antenna needs to be near the plasma
 - Frequently creates ions with energies in the MeV range mainly perpendicular to \vec{B}
 - Often predominant electron heating through collisions with accelerated ions, but can provide ion heating in the right circumstances
 - Some potential for current profile control

OK that's it!

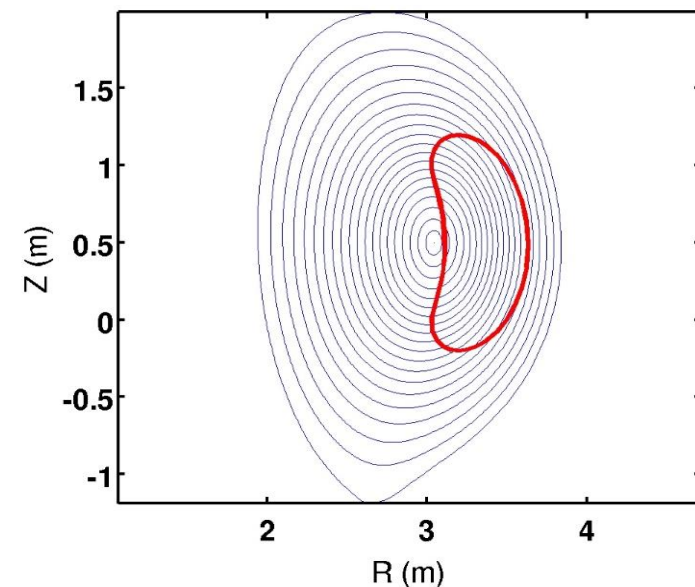
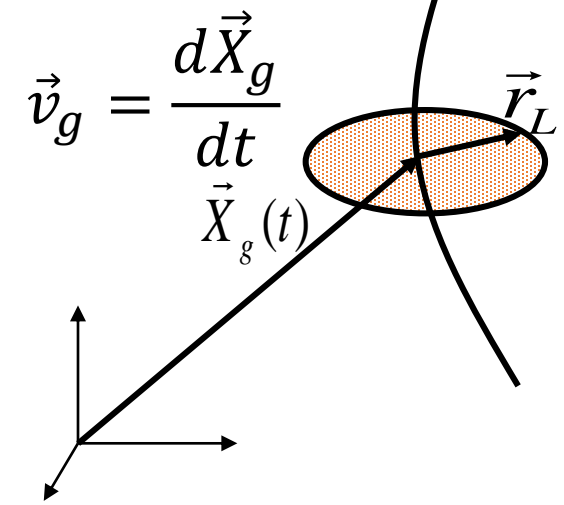
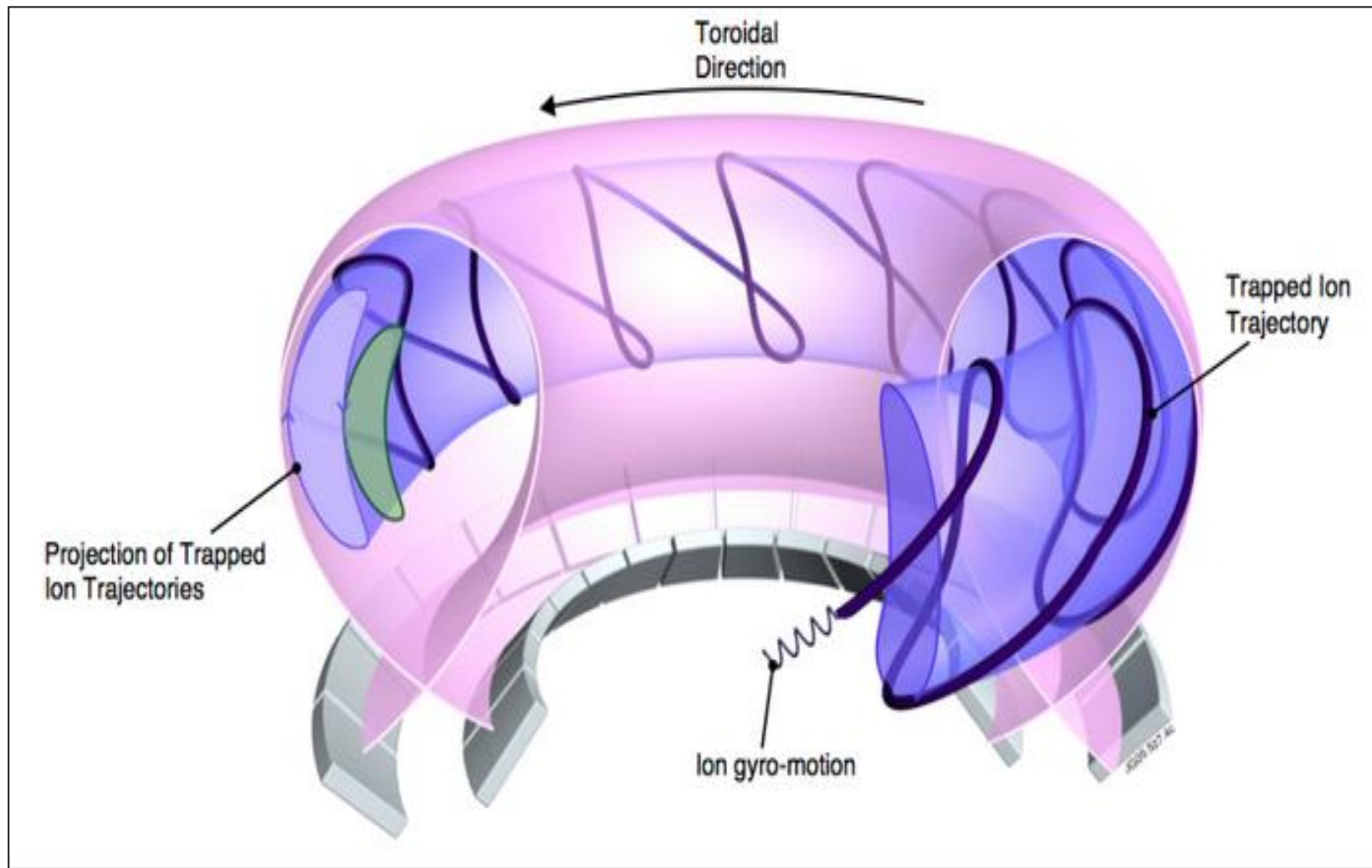
Outline

- Fast ion orbits
- The orbit averaged Fokker-Planck equation
- Alpha particles and classical slowing down
- Neutral Beam injection, the basics
- Ion Cyclotron Resonance Frequency (ICRF) Heating, wave propagation and wave-particle interaction.

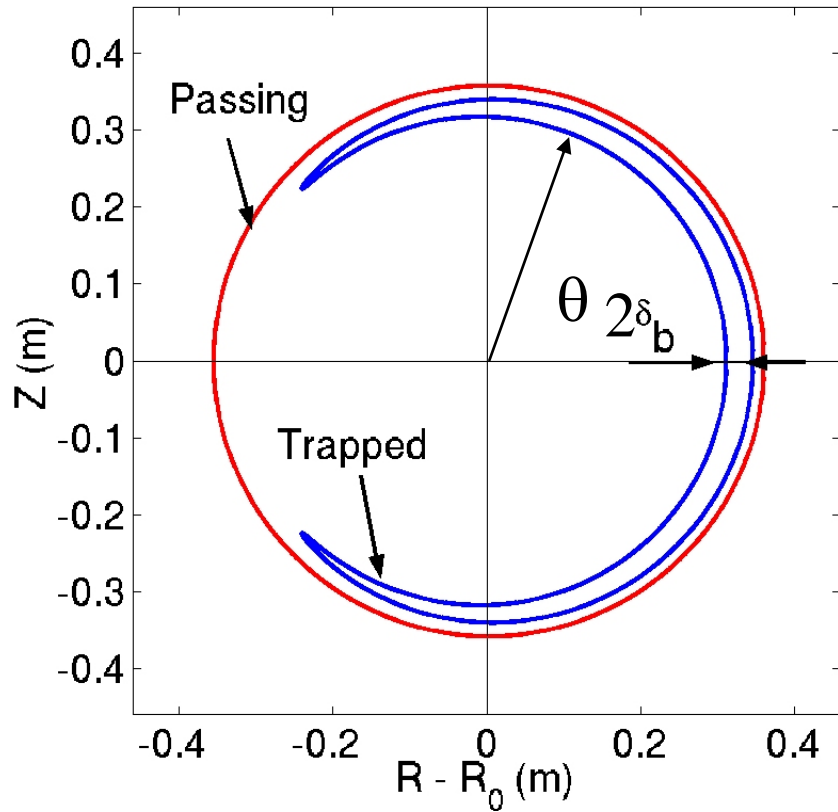
Energetic particle orbits in a tokamak

- Energetic (or fast) ions with $v \gg v_{th}$ have $\tau_b/t_{coll} \ll 1$

Guiding centre orbit



Standard, small width, banana orbits

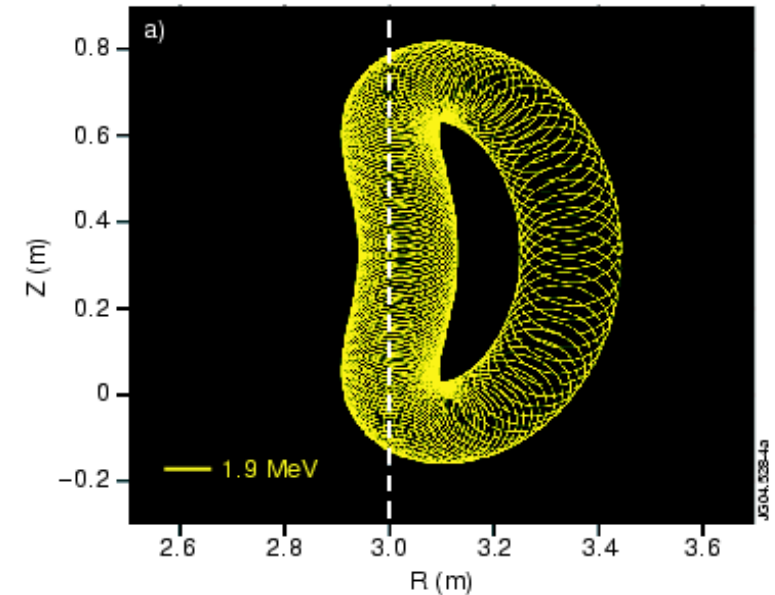


$$\delta_b \sim \frac{v_{\parallel 0}}{\omega_{B\theta}} = \frac{v}{\omega_c} q \left(\frac{r}{R} \right)^{-1/2}$$

- Take 3.5 MeV alpha particle; $q=1$; $r/R=0.1$
- ITER $2\delta_b/a \sim 0.2$
- JET $2\delta_b/a \sim 0.6$

Non-standard orbits when $2\delta_b \gtrsim r$

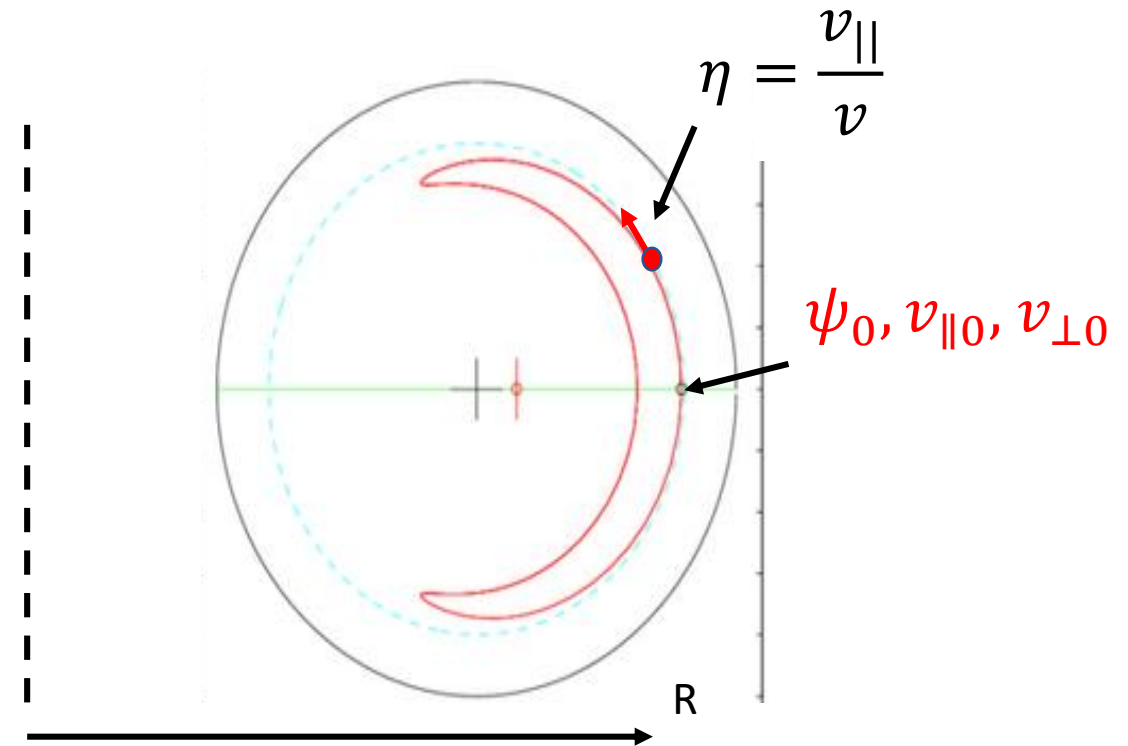
1.9 MeV alpha particle in JET: $2\delta_b \approx 0.4m$



Crossed border into non standard regime

Invariants

- How many variables, invariants, are needed to identify an orbit?
- Answer: three! E.g.
 1. Label to position along the orbit where $B = B_{min}$ by the poloidal flux function $\psi = \psi_0$
 2. The flux ψ_0 together with the parallel and perpendicular velocities, $v_{\parallel 0}$ and $v_{\perp 0}$, then defines an orbit



The poloidal flux function is $\psi = \int_0^r R B_{\theta} dr$ (integration along midplane)

An often used set of Invariants

$$W = \frac{1}{2} m v^2 \quad \text{or} \quad v \quad \text{Energy or Velocity}$$

$$\Lambda = \frac{\mu B_0}{E} = \frac{v_{\perp}^2}{v^2} \frac{B_0}{B} \quad \text{Magnetic momentum / Energy}$$

$$P_{\varphi} = m R v_{\parallel} \frac{B_{\varphi}}{B} + Z e \psi \quad \text{Toroidal canonical (or angular) momentum}$$

ψ is the poloidal flux function (for a circular tokamak $\psi = \int_0^r R B_{\theta} d\eta$)

- There are regions in $(E, \Lambda, P_{\varphi})$ corresponding to two orbits, which we distinguish by $\sigma = \pm 1$
- Note that for trapped ions: $P_{\varphi} = Z e \psi \Big|_{v_{\parallel}=0}$

Orbit equation (approximate)

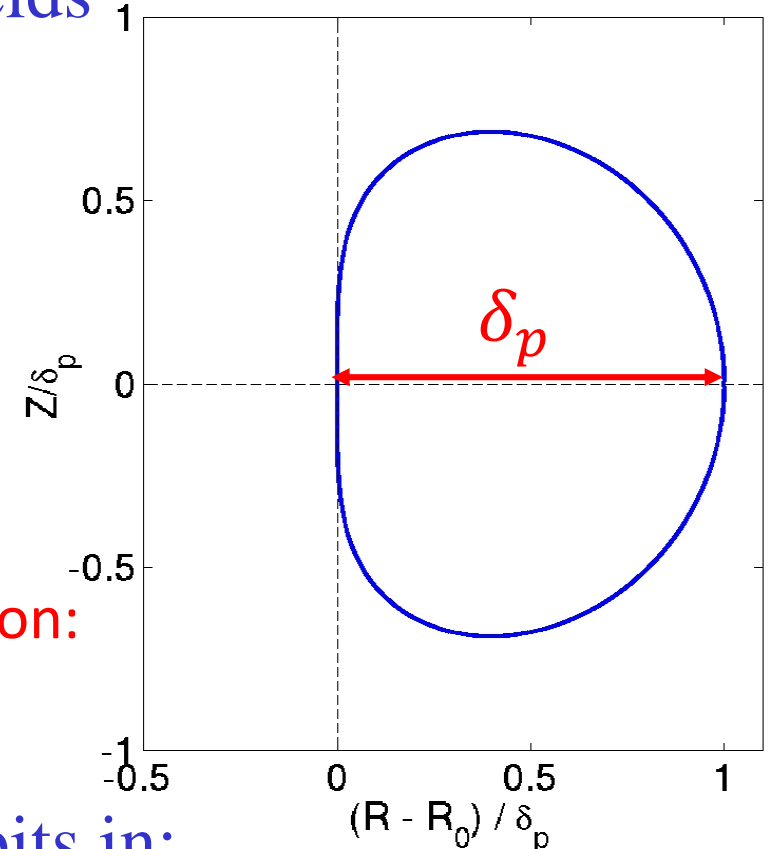
- Equating $v_{||}$ from P_{φ} with $v_{||} = \pm v \sqrt{1 - \Lambda B/B_0}$ yields

$$\pm \sqrt{1 - \Lambda \frac{B}{B_0}} \approx \frac{1}{mR_0 v} [-Ze\psi + P_{\varphi}]$$

- Consider a typical “Potato orbit” with $v_{||} = 0$ at the magnetic axis, $\rightarrow \Lambda = 1, P_{\varphi} = 0$

$$\delta_p \approx \left(\frac{2qv}{R_0 \omega_c} \right)^{2/3} R_0$$

Typical for a JET 1 MeV hydrogen ion:
 $\delta_p = 30 \text{ cm}$



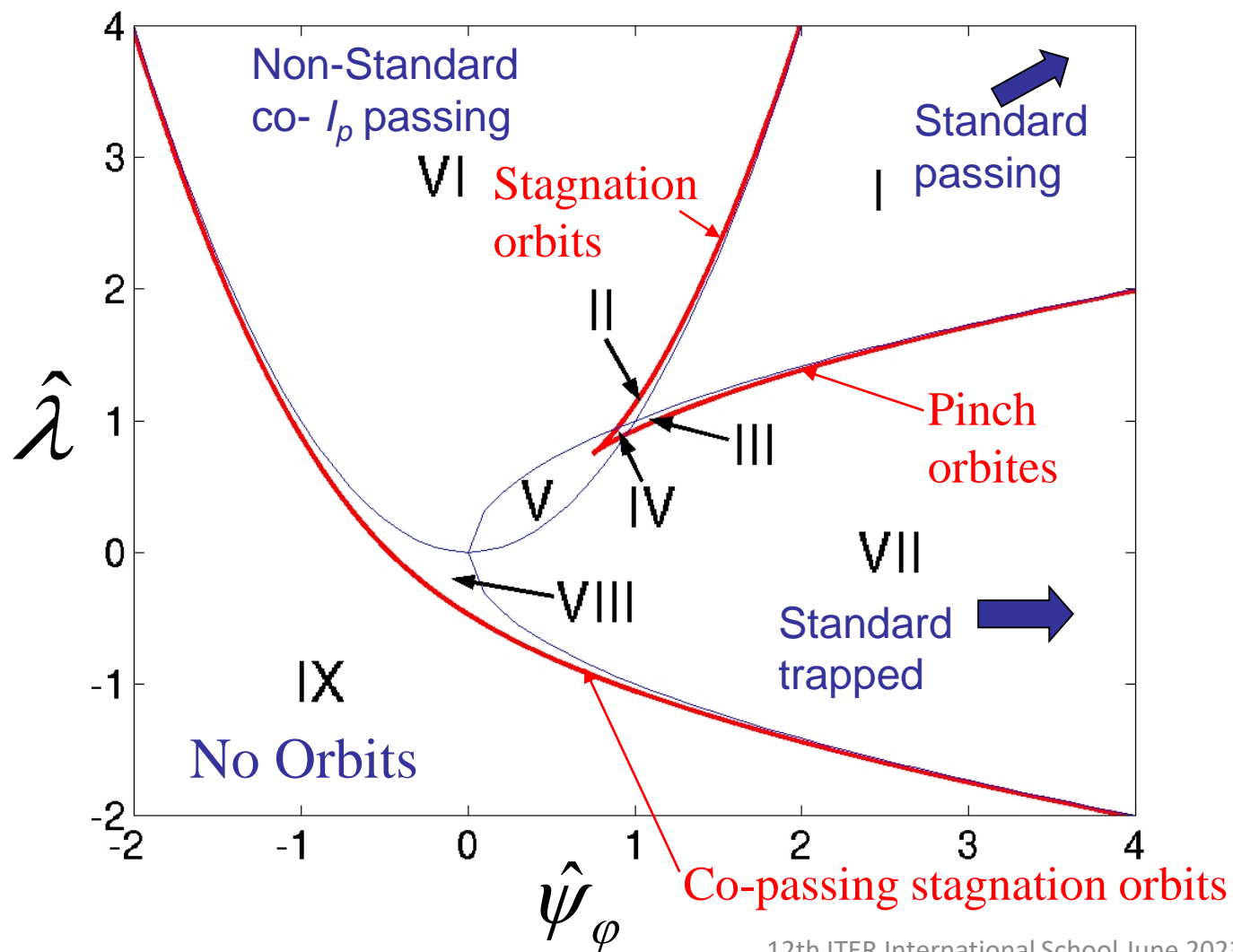
- The equation was the basis for the classification of orbits in:
 L.-G. Eriksson and F. Porcelli, PPCF, **43**, R145 (2001)

- Several authors have classified orbits see e.g. J. Egedal 2000 Nucl. Fusion **40** 1597

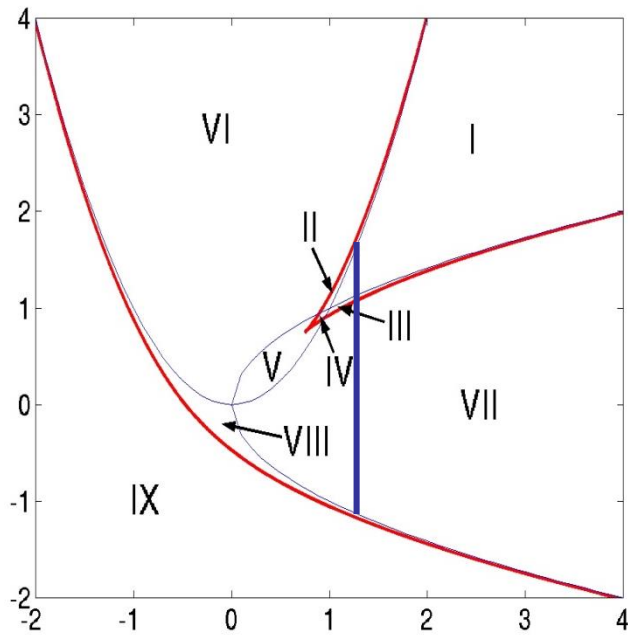
Orbit classification

Low β tokamak, circular flux surfaces; $\hat{\lambda} = \left(1 - \frac{1}{\Lambda}\right) \frac{R_0}{\delta_p}$, $\hat{\psi}_\varphi = \frac{2q}{ZeB_0\delta_p^2} P_\varphi$

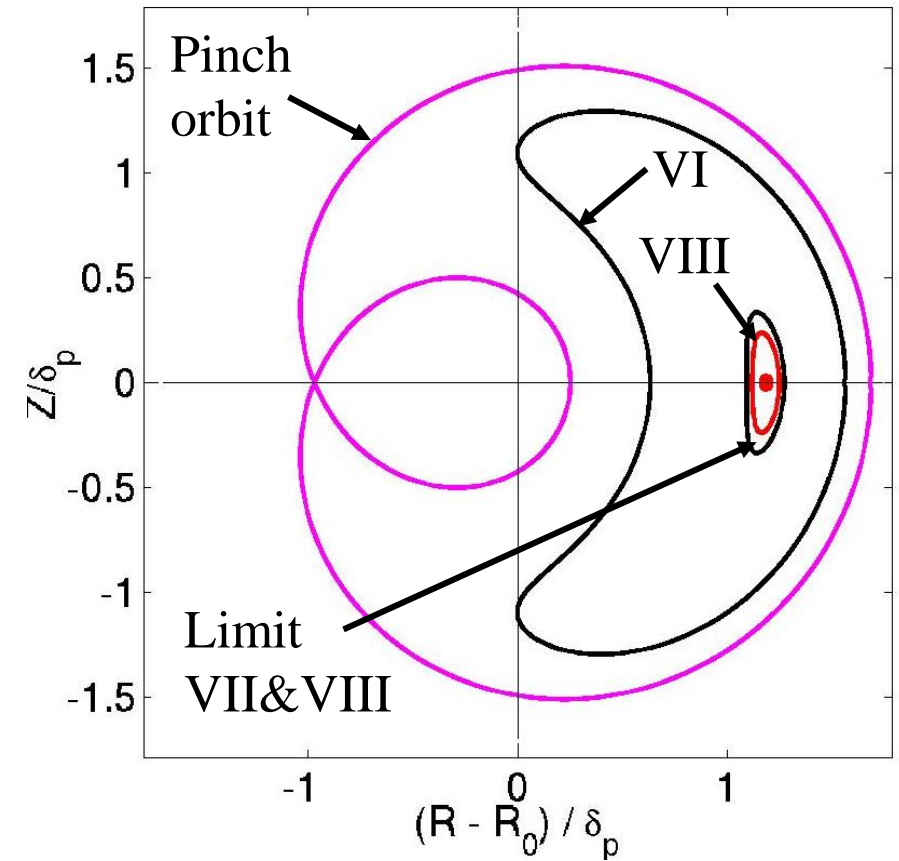
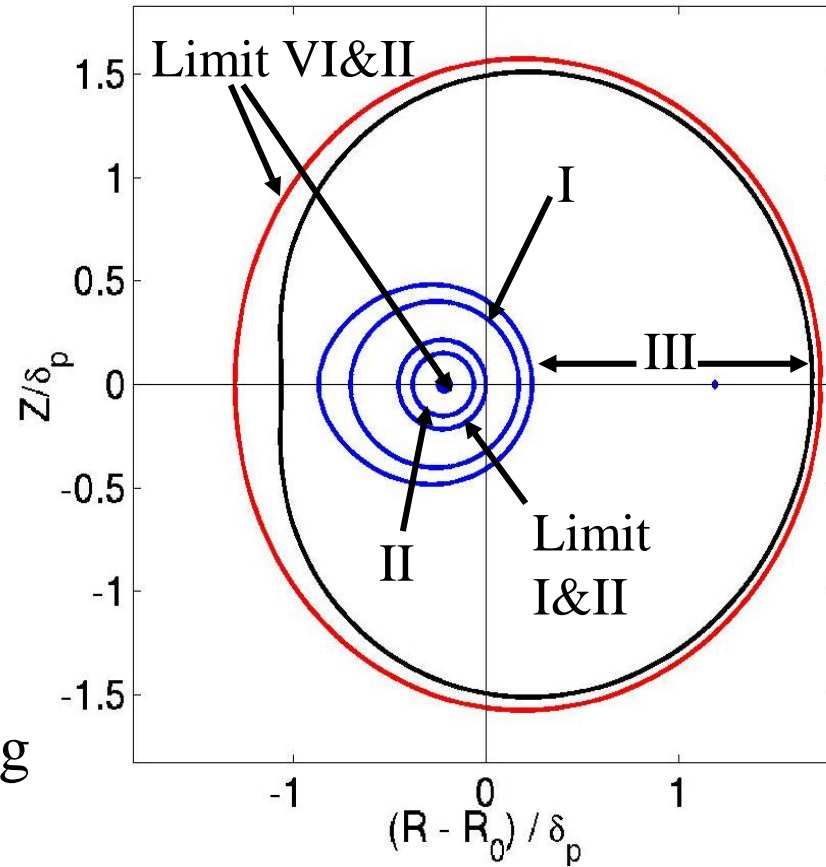
Reg.	Orbits
I	two
II	two
III	two
IV	two
V	one
VI	one
VII	one
VIII	one
IX	none



Orbits for $\hat{\psi}_\phi = 1.2$



- Co-passing
- Counter-passing
- Trapped



The orbit averaged Fokker-Planck equation

- In order to analyse fast ions we will need the Fokker-Planck equation

$$\vec{z} = (\vec{r}, \vec{v})$$

$$\frac{\partial f}{\partial t} + \dot{z}^i \frac{\partial f}{\partial z^i} = C(f) + S - L(f) + Q(f)$$

Collision operator Source term Loss term

↑ Particle orbits enter here ↙ Wave particle interaction

- We are mainly interested in evolution of f on the collisional time scale
- It is possible to reduce this 6D F-P equation to 3D by orbit averaging:

Intermediate variables $\vec{z} = (\vec{r}, \vec{v}) \rightarrow (\vec{J}, \vec{\theta})$

- The motion of a single particle in an axisymmetric torus is integrable \rightarrow there exist a canonical transformation to action angle variables ¹ $(\vec{J}, \vec{\theta})$
- In these the (unperturbed) Hamiltonian H_0 depends only on the actions:

$$H_0 = H_0(\vec{J}) \quad \longrightarrow \quad \Omega^i = \dot{\theta}_0^i = \frac{\partial H_0}{\partial J^i}$$

$$J^1 = \frac{m\mu}{Ze} = \frac{\Lambda E}{\omega_{c0}}, \quad J^3 = P_\varphi = mRv_{||} \frac{B_\varphi}{B} + Ze\psi, \quad J^2 \sim \text{Toroidal flux enclosed by a poloidal orbit}$$

- Roughly speaking the angles describe:

θ^1 Position in the Larmor rotation

θ^2 Position along the poloidal guiding centre orbit

θ^3 Toroidal position of banana centre

$$\Omega^1 = \langle \omega_c \rangle$$

$$\Omega^2 = 2\pi / \tau_b$$

$$\Omega^3 = \langle \dot{\varphi} \rangle$$

$$\langle \dots \rangle = \text{Orbit average}$$

¹A.N. Kaufman Phys. Fluids. 1972.

- Fokker-Planck equation after variable transformation $\vec{z} = (\vec{r}, \vec{v}) \rightarrow (\vec{J}, \vec{\theta})$

$$\frac{\partial f}{\partial t} + \dot{\theta}^i \frac{\partial f}{\partial \theta^i} = C(f) + S - L + Q(f)$$

- Multiple time scale expansion: $f = f_0 + \frac{\tau_b}{\tau_c} f_1 + \dots$

- To order -1 we have: $\Omega^i \frac{\partial f_0}{\partial \theta^i} = 0 \rightarrow f_0 = f_0(\vec{J}) = f_0(\vec{I})$

- Define orbit average: $\langle \dots \rangle = (2\pi)^{-3} \iiint_0^{2\pi} (\dots) d^3\theta \approx \frac{1}{\tau_b} \int_0^{\tau_b} (\dots) d\tau$

- The zero order equation yields after applying $\langle \dots \rangle$

$$\frac{\partial f_0}{\partial t} = \langle C(f_0) \rangle + \langle S \rangle - \langle L \rangle + \langle Q(f_0) \rangle$$

- The collision operator is conservative, i.e. it is represented as a divergence of a flow in phase space.

$$C(f) = \frac{1}{v^2} \frac{\partial}{\partial v} \left\{ v^2 \left[\alpha(v)f + \beta(v) \frac{\partial f}{\partial v} \right] \right\} + \frac{\partial}{\partial \eta} \left[\gamma(v)(1 - \eta^2) \frac{\partial f}{\partial \eta} \right]$$

Slowing down
Energy diffusion
Pitch angle scattering

- For test particles it simplifies remarkably in the high energy limit $v \gg v_{th}$

Slowing down on el.

Slowing down on ions

$$C(f) = C_{s.d.}(f) + C_{p.a.s}(f) = \frac{1}{v^2} \frac{\partial}{\partial v} \left[v^2 \frac{v + \frac{v_c^3}{v^2}}{t_s} f \right] + \frac{\partial}{\partial \eta} \left[\frac{1}{t_s} \frac{v_\gamma^3}{4v^3} (1 - \eta^2) \frac{\partial f}{\partial \eta} \right]$$

Slowing down term

Pitch angle scattering

t_s is the ion electron slowing down time; v_c is the critical velocity; v_γ is a characteristic velocity for pitch angle scattering

Slowing down time and characteristic energies of $C(f)$

- Example DT plasma $n_e = 1 \cdot 10^{20} m^{-3}$

Slowing down time:

$$t_s \approx 6.27 \times 10^{14} \frac{AT_{e,eV}^{3/2}}{Z^2 n_e \ln \Lambda}$$

Critical Energy:

$$\frac{1}{2} m v_c^2 = 14.8 k T_e A \left[\frac{1}{n_e} \sum_j \frac{n_j Z_j^2}{A_j} \right]^{2/3}$$

Char. Energy for pitch angle scatt.:

$$\frac{1}{2} m v_\gamma^2 = 14.8 k T_e [2A^{1/2} Z_{eff}]^{2/3}$$

	D		⁴ He	
	$T_e = 10$ keV	$T_e = 20$ keV	$T_e = 10$ keV	$T_e = 20$ keV
t_s	0.7s	2s	0.35s	1s
$\frac{1}{2} m v_c^2$	165 keV	330 keV	330 keV	660 keV
$\frac{1}{2} m v_\gamma^2$	296 keV	592 keV	373 keV	746 keV

High energy orbit averaged $C(f_0)$

- Using tensor transformation rules, we obtain for the invariants $\vec{I} = (v, \Lambda, P_\varphi)$

$$\langle C(f_0) \rangle = \frac{1}{\sqrt{g}} \frac{\partial}{\partial v} \left[\sqrt{g} \left\langle \frac{v + \frac{v_c^3}{v^2}}{t_s} \right\rangle f_0 \right] + \frac{1}{\sqrt{g}} \frac{\partial}{\partial P_\varphi} \left[\sqrt{g} \left\langle mR \frac{B_\varphi}{B} \eta \frac{v + \frac{v_c^3}{v^2}}{t_s} \right\rangle f_0 \right] +$$

$$\frac{1}{\sqrt{g}} \frac{\partial}{\partial I^i} \left[\sqrt{g} \left\langle \frac{1}{t_s} \frac{v_\gamma^3}{4v^3} (1 - \eta^2) \frac{\partial I^i}{\partial \eta} \frac{\partial I^j}{\partial \eta} \right\rangle \frac{\partial f_0}{\partial I^j} \right]$$

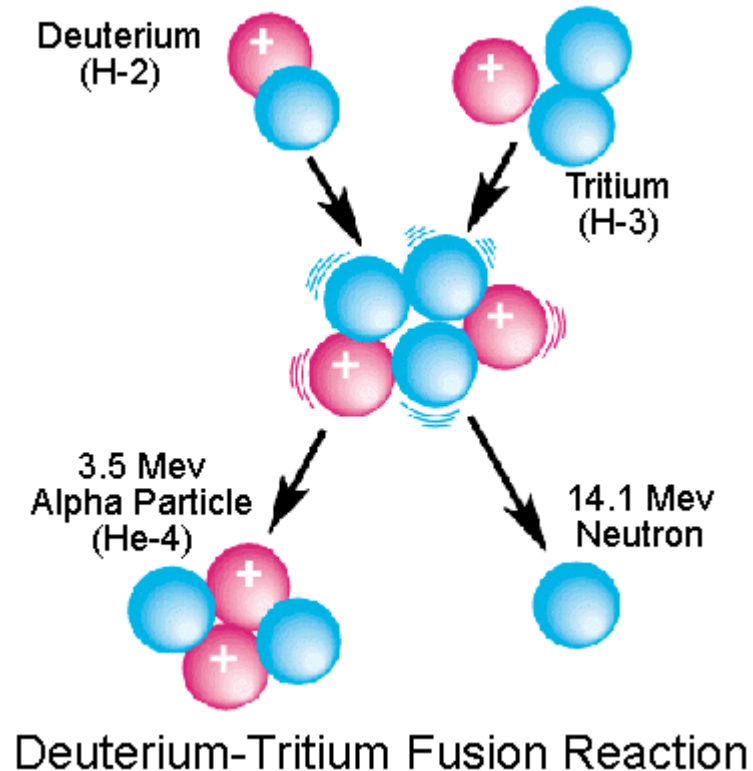
$$\sqrt{g} = \left| \frac{\partial \vec{z}}{\partial (\vec{I}, \vec{\theta})} \right| = \frac{v^3 \tau_b}{4\pi m \omega_{c0}}$$

Methods for solution of the orbit averaged Fokker-Planck equation

- Direct solution with finite difference scheme (very intricate and impressive)
 - *F. S. Zaitsev; M. R. O'Brien; M. Cox Physics of Fluids B: Plasma Physics* 5, 509–519 (1993).
 - Yu V Petrov and R W Harvey 2016 *Plasma Phys. Control. Fusion* **58** 115001 (including QL operator; **very impressive**)
- Direct solution with a Monte Carlo algorithm
 - *Carlsson J., et al Proceedings of the Joint Varenna-Lausanne Workshop "Theory of Fusion Plasmas", Bologna: Editrice Compositori, 1994, p. 351. (FIDO code)*
- Orbit following Monte Carlo codes with an acceleration scheme
 - A. Pankin, D. McCune, R. Andre et al., *Computer Physics Communications* Vol. 159, No. 3 (2004) 157-184. (NUBEAM used in TRANSP)
 - Seppo Sipilä *et al* 2021 *Nucl. Fusion* **61** 086026 (ASCOT code augmented with ICRF module)

Alpha particles and classical slowing down

- Alpha particles are born in D-T fusion reactions:
 $D+T \rightarrow He^4 (3.52 \text{ MeV}) + n (14.06 \text{ MeV})$.
- The alpha particles slows down by collisions with the bulk plasma, and should sustain the plasma temperature and thereby the fusion burn in a reactor.
- Hence, the confinement of alpha particles is crucial.
- To analyse fusion alpha particles with with the Fokker-Planck equation we need the source: $S(\vec{r}, \vec{v})$



- The spectrum of the alpha particle source depends on the distribution function of the reacting species $S = S(f_D, f_T)$
- The kinematics of the alpha particle producing process dictates*:

$$E_\alpha = \frac{1}{2} m_\alpha V^2 + \frac{m_n}{m_n + m_\alpha} (Q + K) + V \sin \varphi \sqrt{\frac{2m_n m_\alpha}{m_n + m_\alpha} (Q + K)}$$

$$Q \approx 17.49 \text{ MeV}$$

$$K = \frac{1}{2} \frac{m_D m_T}{m_D + m_T} (\vec{v}_D - \vec{v}_T)^2$$

V is the centre of mass velocity of the reacting D and T ions
 φ is the angle between \vec{v}_α and \vec{V} in the centre of mass frame

*see e.g. H Brysk. Plasma Physics, 15(7):611, 1973

- If the deuterium and tritium ions have thermal distributions (i.e. are Maxwell distributed) the source term can be approximated by*,

$$\langle S \rangle = \frac{\langle \sigma v \rangle_{DT}}{C_{norm}} e^{-\frac{5m_{\alpha}^2(v^2 - v_{\alpha 0}^2)^2}{64T_i E_{\alpha 0}}}$$

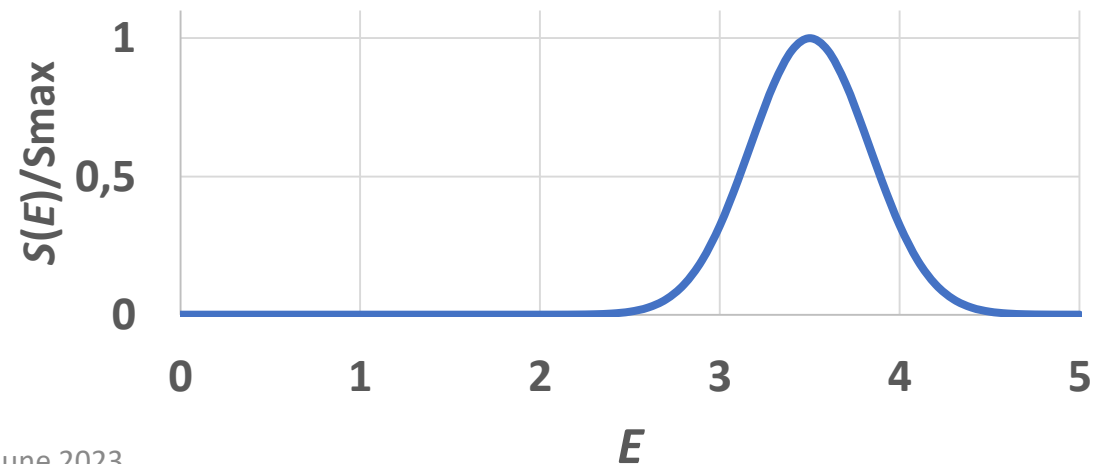
$$C_{norm} = \int_0^{\infty} e^{-\frac{5m_{\alpha}^2(v^2 - v_{\alpha 0}^2)^2}{64T_i E_{\alpha 0}}} 4\pi v^2 dv$$

$$E_{\alpha 0} = \frac{1}{2} m_{\alpha} v_{\alpha 0}^2 = 3.5 MeV$$

- The full width at half maximum FWHM of the alpha particle source, $\Delta E_{\alpha FWHM}$, is: $\Delta E_{\alpha FWHM} (MeV) = 0.088 \sqrt{T_i (KeV)}$

- Typical for ITER would be $T_i = 20 keV$

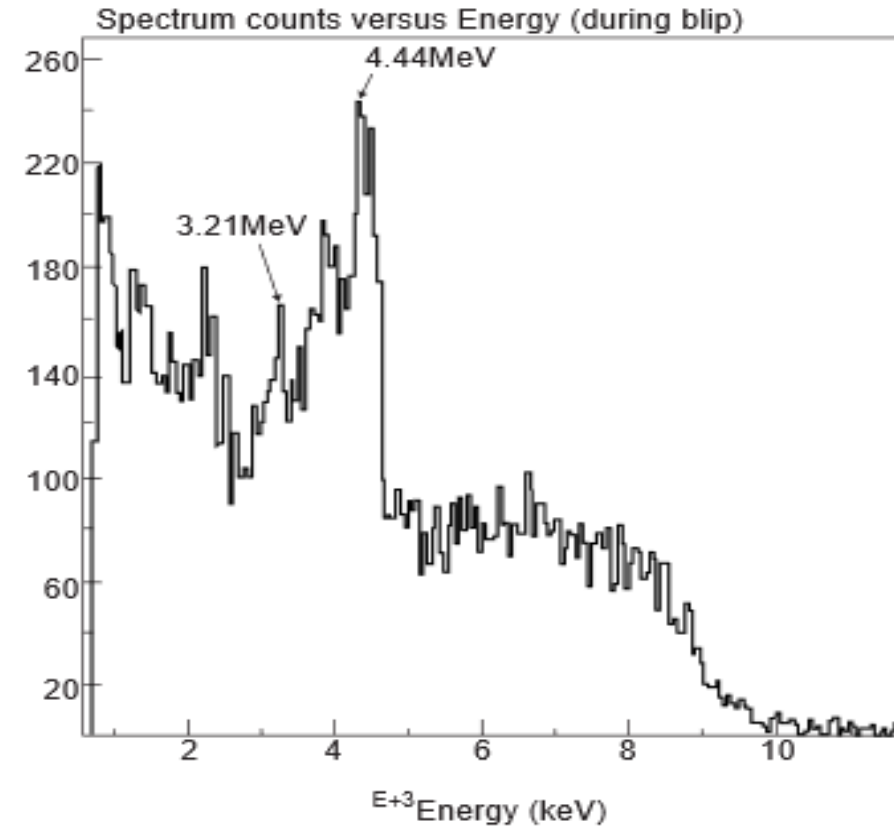
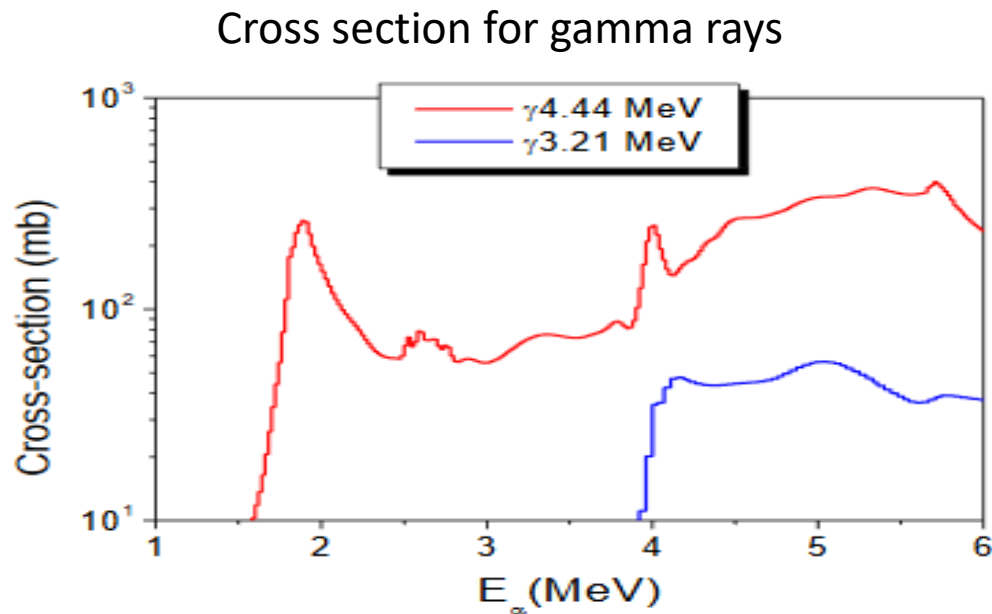
➔ $\Delta E_{\alpha FWHM} = 0.39 MeV$



*see e.g. H Brysk. Plasma Physics, 15(7):611, 1973

γ ray measurement at JET during trace tritium campaign in 2003

Interaction between α's and Be impurity
 → excitation of C^{13} and following de-excitations to γ rays



Deuterium plasma with 15 MW of NBI injected at 105 KeV and 1.5 MW of Tritium blip; $T_i(0) = 6$ keV and $n_e = 6 \cdot 10^{10} \text{ m}^{-3}$

Kiptily et al. Phys Rev Letter 93(2004) 115001

- To make analytic progress, we turn to the simplest form of the Fokker-Planck equation: small banana width and an isotropic alpha particle source

$$\frac{\partial f_{\alpha 0}}{\partial t} = \frac{1}{v^2} \frac{\partial}{\partial v} \left[v^2 \frac{v + \frac{v_c^3}{v^2}}{t_s} f_{\alpha} \right] + \frac{\langle \sigma v \rangle_{DT}}{4\pi v^2} \delta(v_{\alpha} - v_{\alpha 0})$$

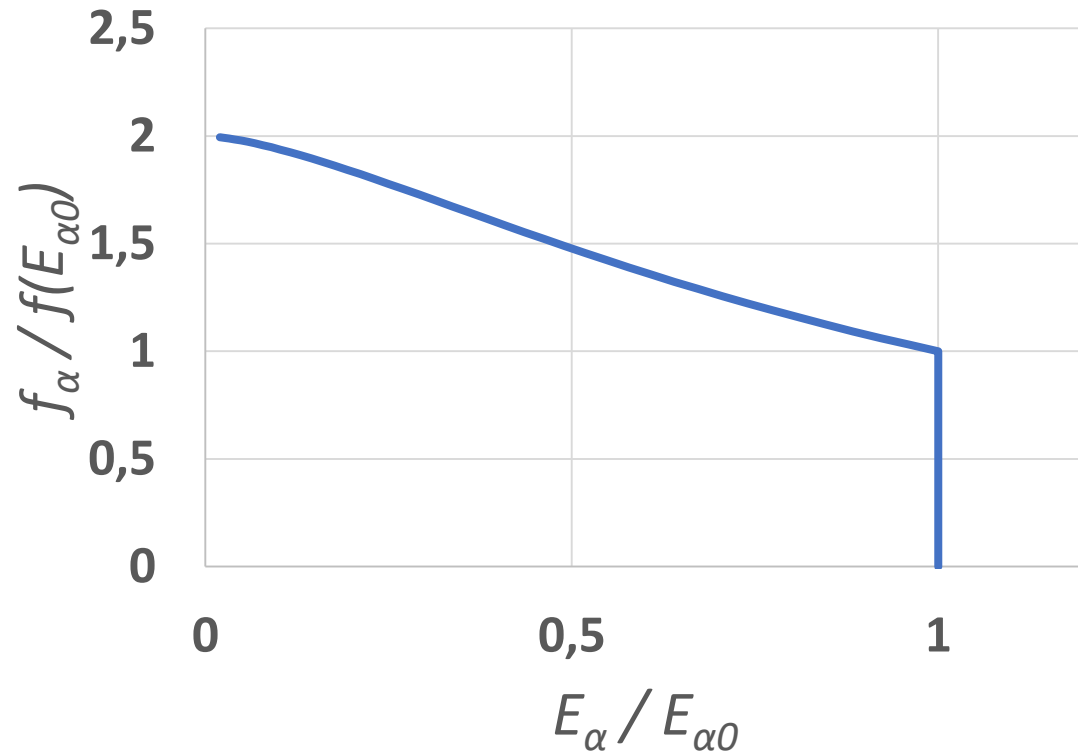
- In steady state the equation is easy to integrate and we get,

$$f_{\alpha 0}(v, \vec{r}) = \frac{t_s \langle \sigma v \rangle_{DT}}{4\pi v_{\alpha 0}^2} \frac{H(v - v_{\alpha 0})}{v^3 + v_c^3}$$

(H is the Heaviside step function)

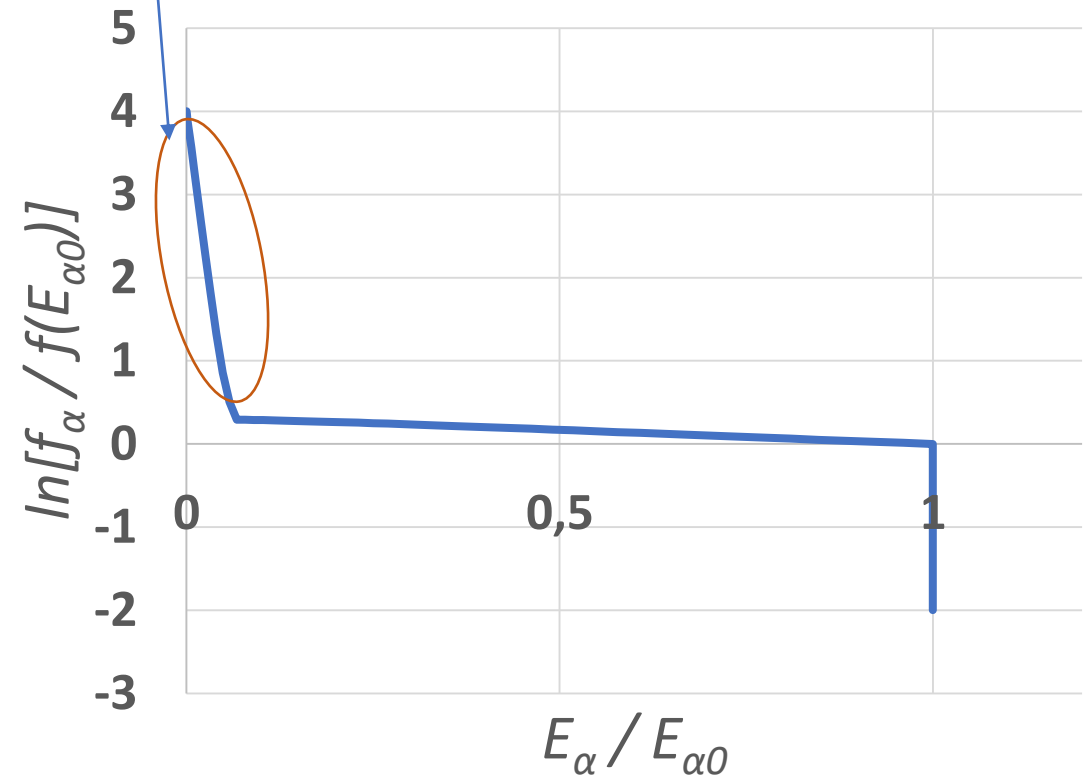
- This is the classical slowing down distribution

Alpha particle slowing down distribution



Thermal Maxwell distribution

Alpha particle distribution



- The collisional power density to the electrons is given by

$$p_e = - \int_0^\infty \frac{m_\alpha}{2} v^2 C_{s.d,e}(f_{\alpha 0}) 4\pi v^2 dv = - \int_0^\infty \frac{m_\alpha}{2} v^2 \frac{1}{v^2} \frac{\partial}{\partial v} \left(v^2 \frac{v}{t_s} f_{\alpha 0} \right) 4\pi v^2 dv_\alpha$$

- With the classical slowing down distribution the integral is doable,

$$p_e = p_\alpha \left[1 - \frac{2v_c^2}{3v_{\alpha 0}^2} \left\{ \frac{1}{2} \ln \left[1 - \frac{v_{\alpha 0}}{v_c} + \left(\frac{v_{\alpha 0}}{v_c} \right)^2 \right] - \ln \left(1 + \frac{v_{\alpha 0}}{v_c} \right) + \sqrt{3} \left[\arctan \left(\frac{\frac{2v_{\alpha 0}}{v_c} - 1}{\sqrt{3}} \right) + \frac{\pi}{6} \right] \right\} \right]$$

With $T = 20 \text{ keV}$ ($\frac{1}{2}mv_c^2 \approx 660\text{keV}$) we obtain

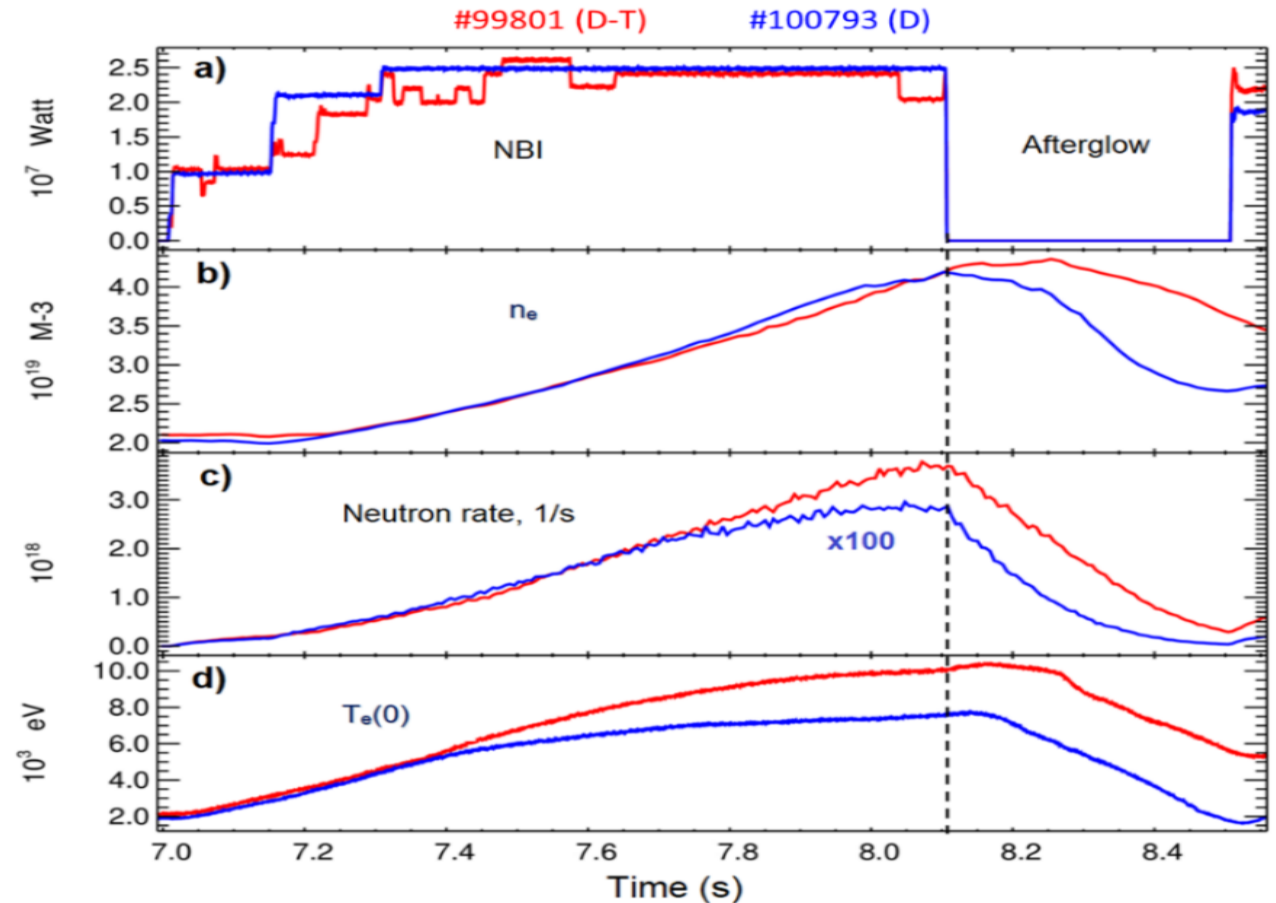
$$p_e \approx 0.7P_\alpha \quad \text{and} \quad p_i \approx 0.3P_\alpha$$

With $T = 10 \text{ keV}$ ($\frac{1}{2}mv_c^2 \approx 330\text{keV}$) we obtain

$$p_e \approx 0.83p_\alpha \quad \text{and} \quad p_i \approx 0.17P_\alpha$$

Alpha particle heating of electrons in JET DTE2

- Hot off the press1!
- “Afterglow” experiments in the recent JET DTE2 experiments demonstrated electron heating by alpha particles.
- The results are consistent with TRANSP simulations.

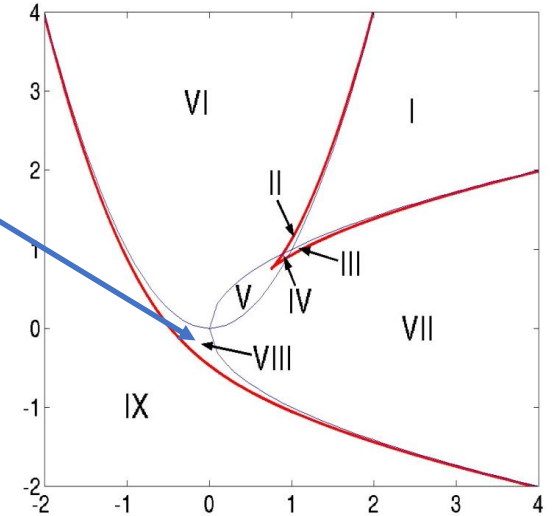


V. Kiptily et al, accepted in Physical Review Letters

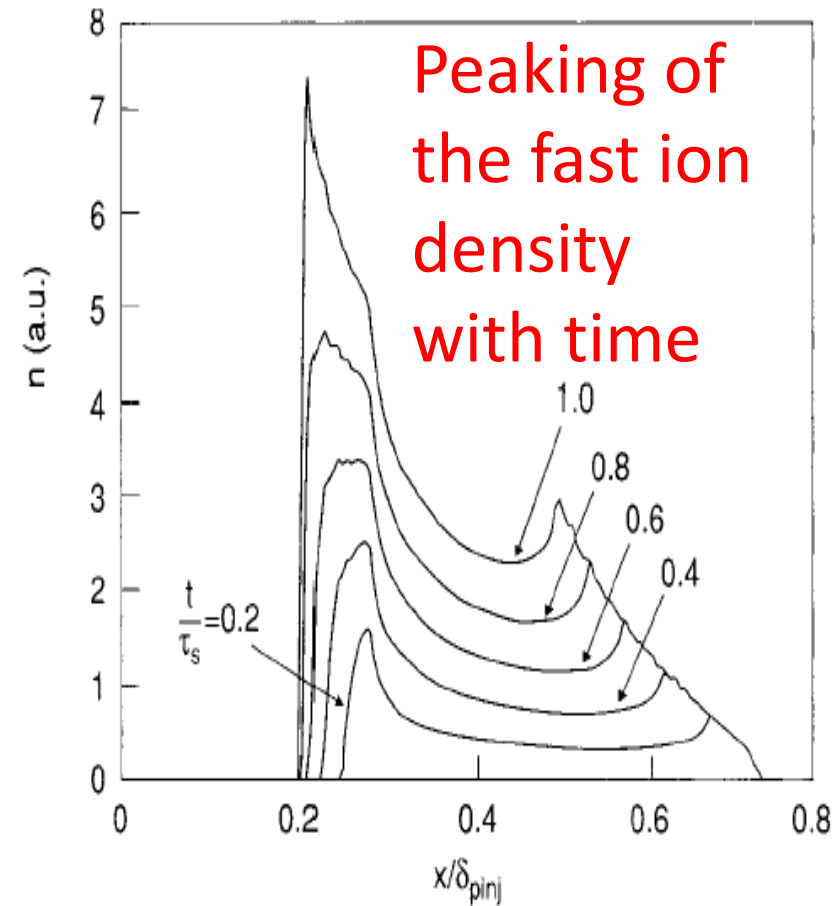
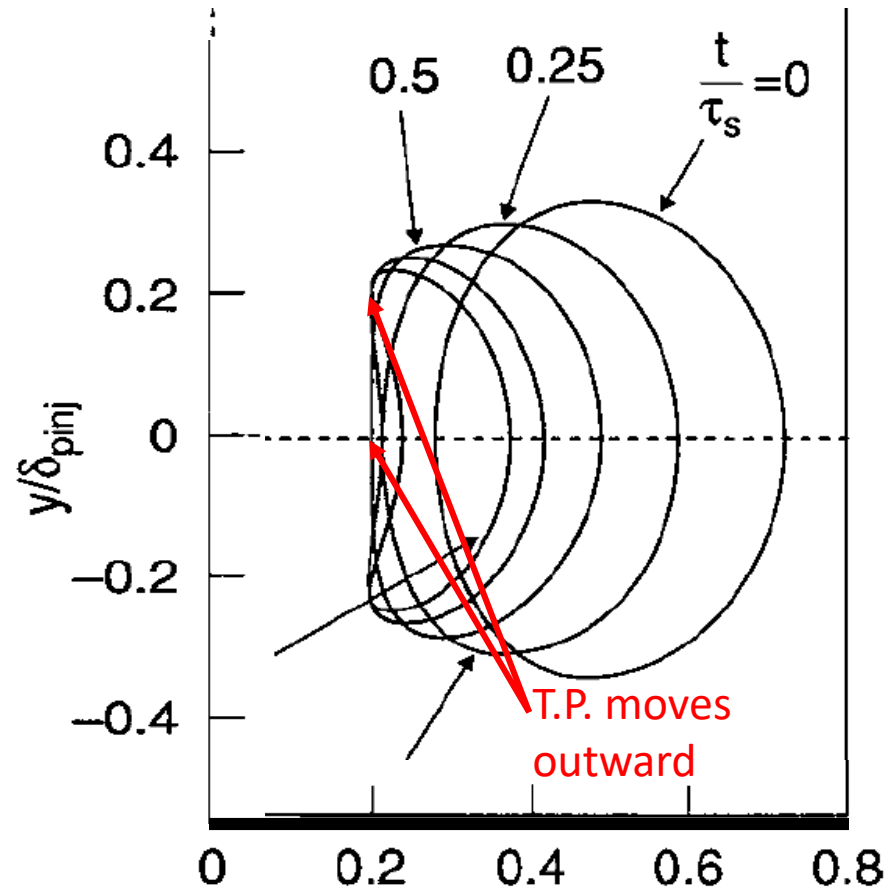
Neoclassical transport due to collisional drag

- For illustration we consider alpha particles borne at the same point in phase space, reg. VIII.
- We can see the solution as a Green's function

$$\begin{aligned}
 \frac{\partial f_0}{\partial t} = & \frac{1}{\sqrt{g}} \frac{\partial}{\partial v} \left[\overbrace{\sqrt{g} \left\langle \frac{v}{t_s} \right\rangle}^{-\Gamma_c^v} f_0 \right] + \frac{1}{\sqrt{g}} \frac{\partial}{\partial P_\varphi} \left[\overbrace{\sqrt{g} \left\langle \frac{P_\varphi - Ze\psi}{t_s} \right\rangle}^{-\Gamma_c^{P_\varphi}} f_0 \right] \\
 & + \bar{S}_0 \delta(v - v_0) \delta(\Lambda - \Lambda_0) \delta(P_\varphi - P_{\varphi 0})
 \end{aligned}$$

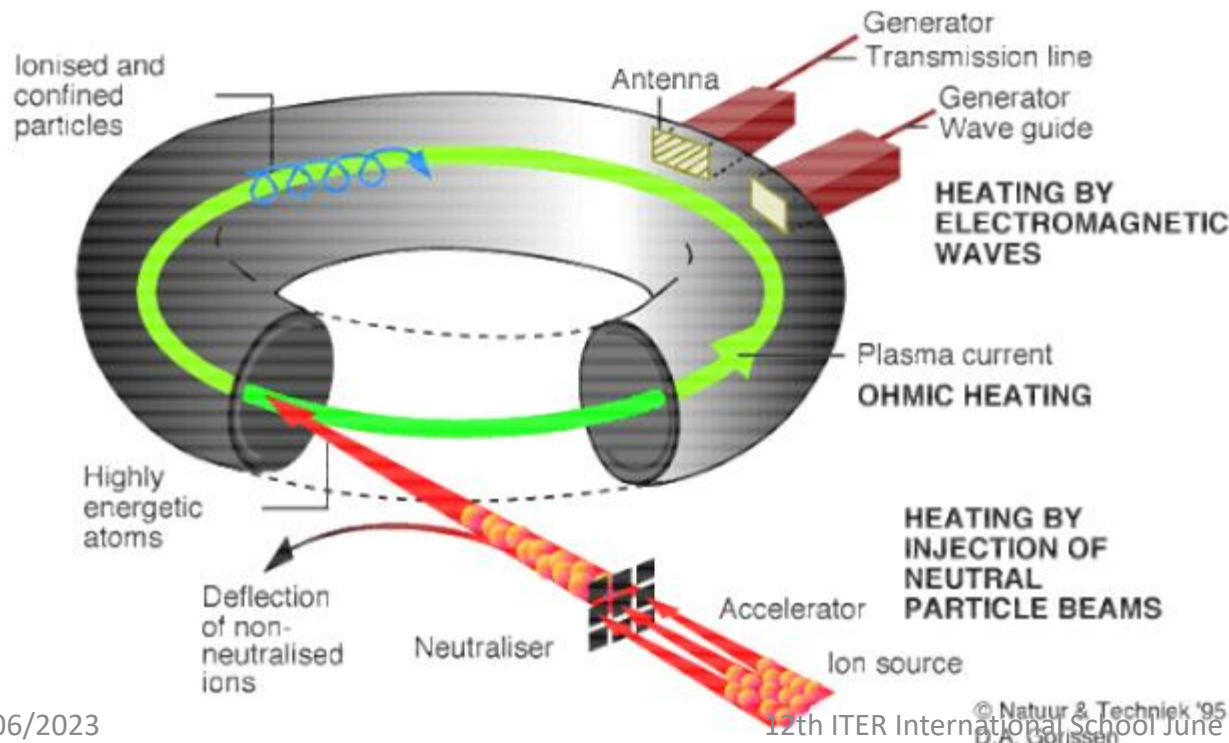


Orbit Evolution and fast ion density



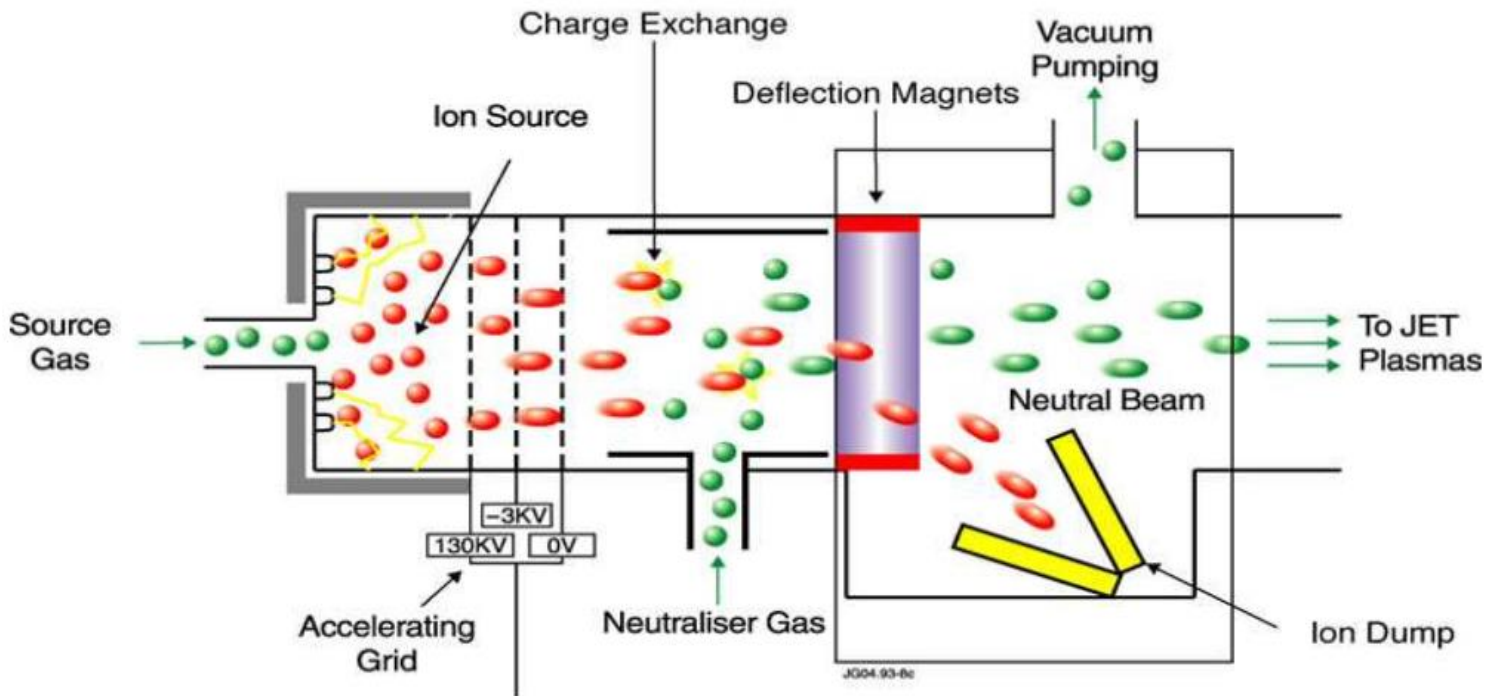
Neutral Beam Injection

- Neutral Beam Injection has been the work horse heating system for many present day tokamaks.
- One the beam particles enter the plasma it is a very robust heating method.
- Like alpha particles, NBI heats the bulk plasma via collisional slowing down



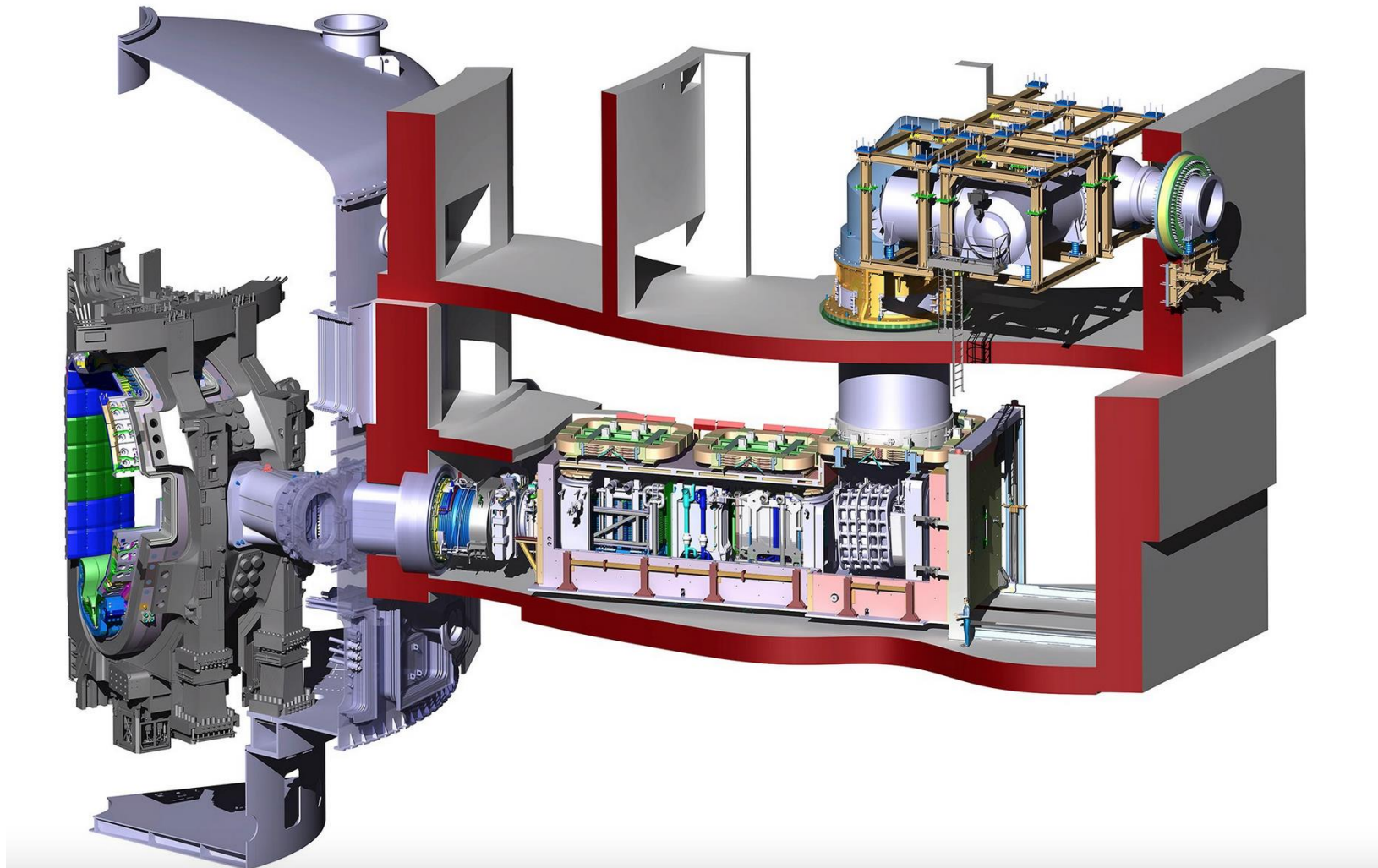
- There are two possibilities:
 - Positive ion sources
 - Negative ion sources
- For injection energies of the order 200 keV and above, the neutralisation efficiency of positive ions is low → negative ion sources are necessary

NBI principle



- Note that for positive ion sources, molecular ions (D₂, D₃) will be in the mix →
- After ionisation in the plasma
 - Full the full injection energy
 - 1/2 injection energy
 - 1/3 injection energy
- Negative ion sources only yield injected ions at the full energy

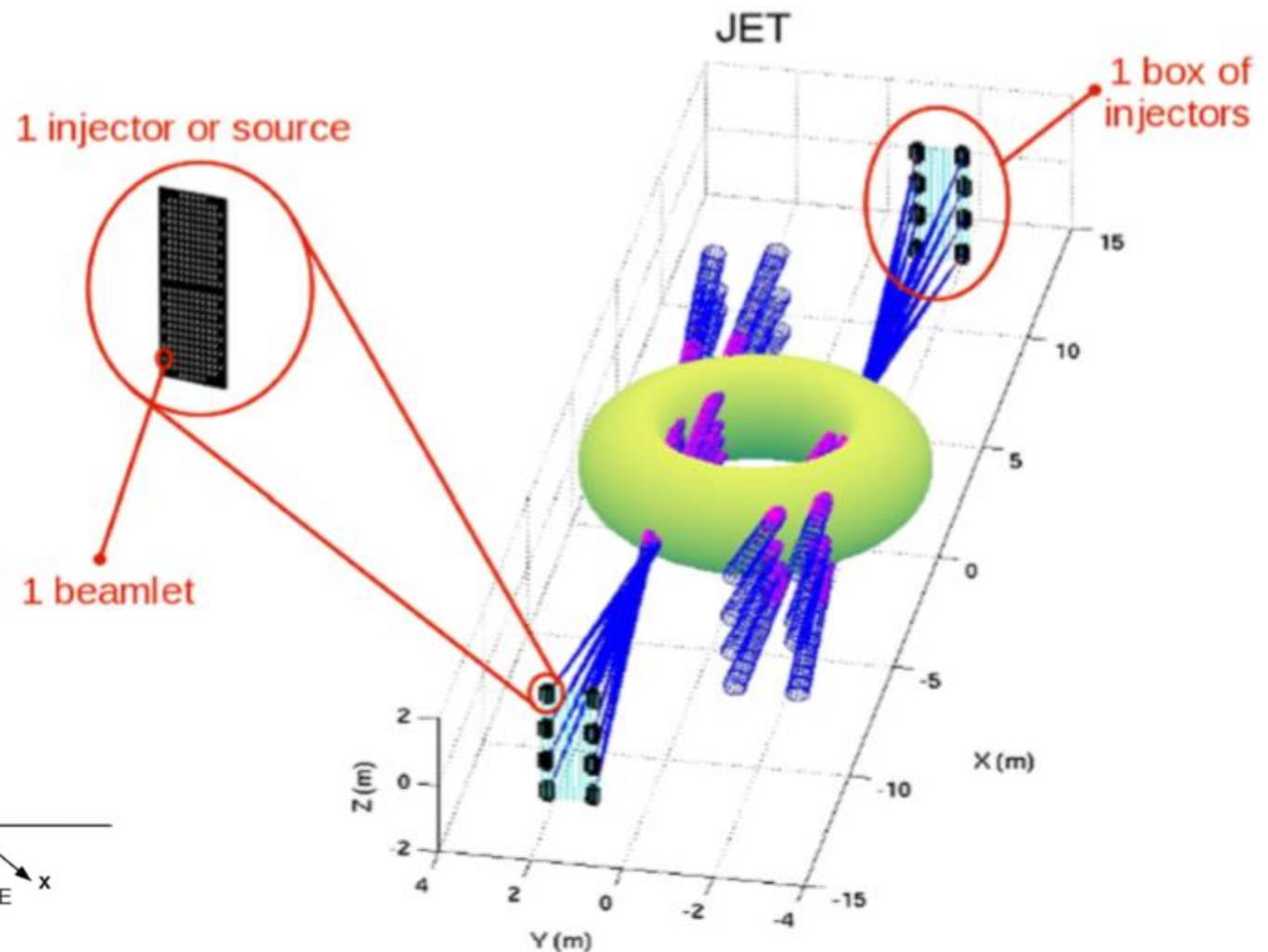
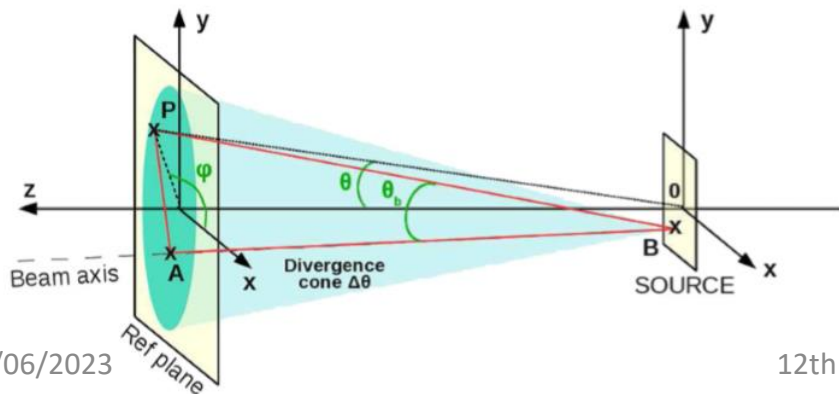
1 Mev negative ion Injector for ITER



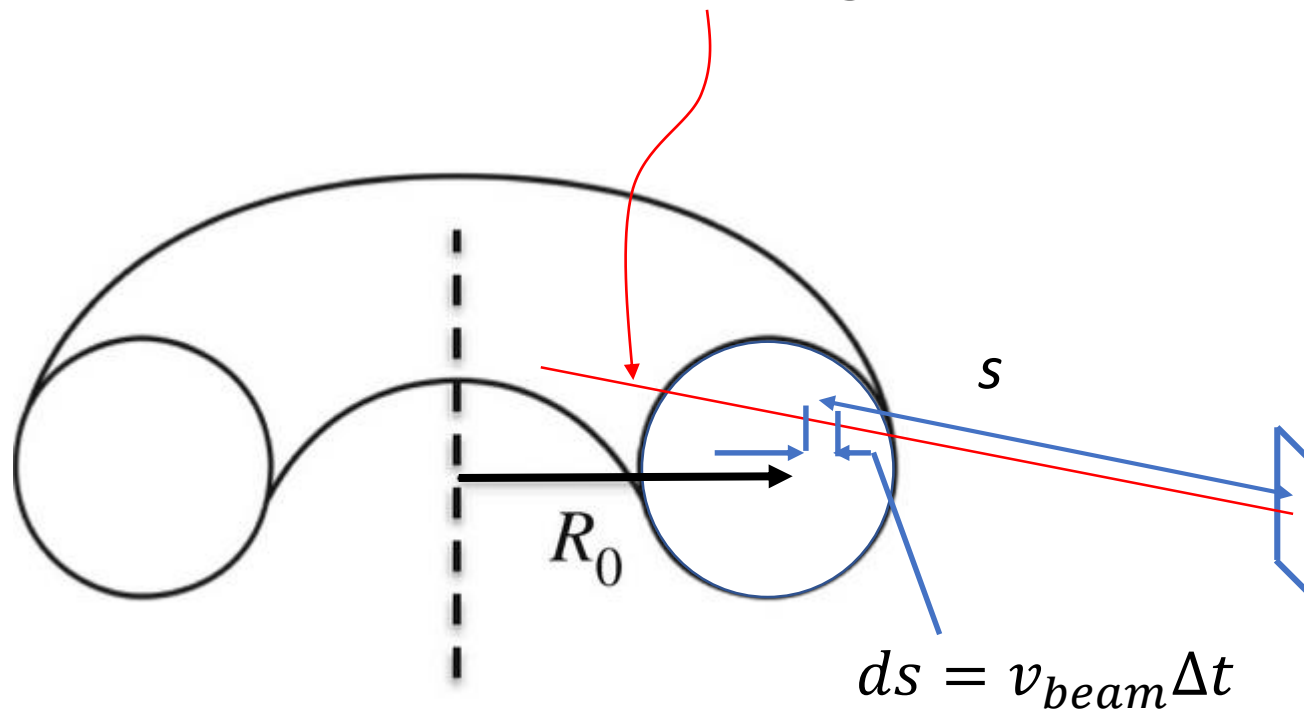
NBI deposition calculation

• In order to calculate deposition profile of NB particles we have to consider:

- Ionisation rate along injection path
- Beam divergence
- Shine trough of the NB particles



- Beam attenuation along a line



$$I = I_0 e^{-\int_0^s \frac{ds}{\lambda}}$$

$$v_{rel} = |\vec{v}_{beam} - v_i|$$

$$\frac{1}{\lambda} = \frac{\langle \sigma_e v_e \rangle n_e}{v_{beam,m}} + \frac{\langle (\sigma_{cx} + \sigma_i) v_{rel} \rangle n_i}{v_{beam,m}} + \sum_{j, imp.} \sigma_j n_j$$

- σ_{ion} and σ_e are the stopping cross-sections collisions with plasma ions and electrons, respectively.
- σ_{cx} is the charge exchange cross-section.
- These can e.g. be found in OPEN-ADAS
- For positive ion NBI “ m ” represents full, 1/2 and 1/3 energy

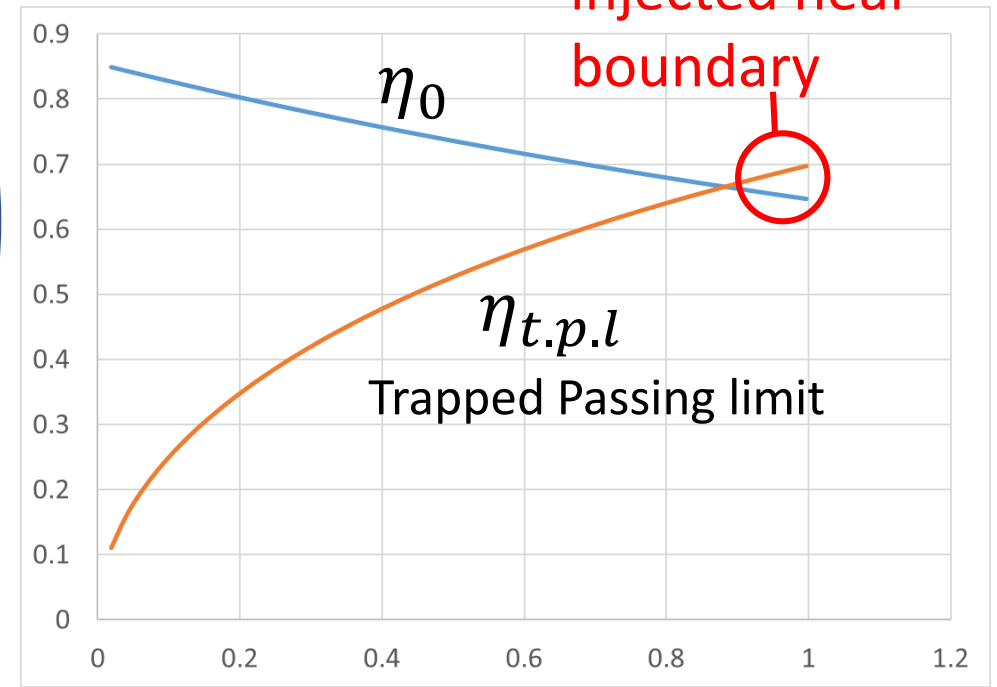
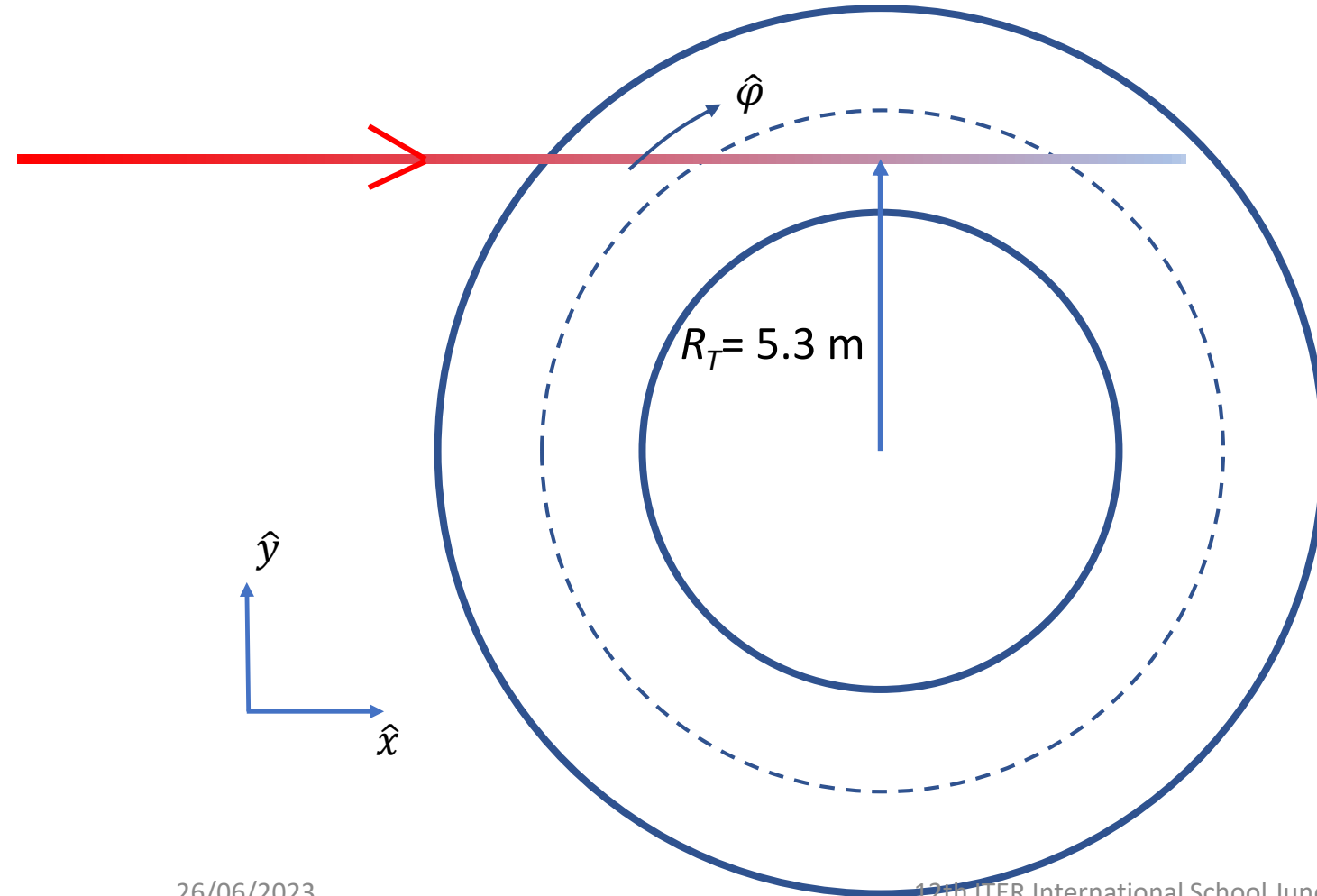
- Methods of calculating NBI deposition
 - Monte Carlo methods
 - NFREYA: R.H. Fowler, J.A. Holmes and J.A. Rome, NFREYA -- A Monte Carlo Beam Deposition Code for Non-circular tokamak Plasmas, Rep. ORNL-TM-6845, Oak Ridge National Laboratory, TN (1979).
 - Model in the TRANSP package NUBEAM
 - BBNBI in the ASCOT package O. Asunta et al Comp. Physcs Comm.188 (2015), 33
 - Narrow beam models
 - Y. Feng et al. Computer Physics Communications **88** (1995) 161
 - M. Schneider et al. Nucl. Fusion 51 (2011) 063019
 - Pencil model
 - P.M. Stubberfield and M.L. Watkins, Experimental department research note DPA(06)87, Multiple Pencil Beam, JET Joint Undertaking, Abingdon, Oxfordshire (1987).

Injection geometry in ITER (very simplified)

- The cosine of the pitch angle is roughly approximated by,

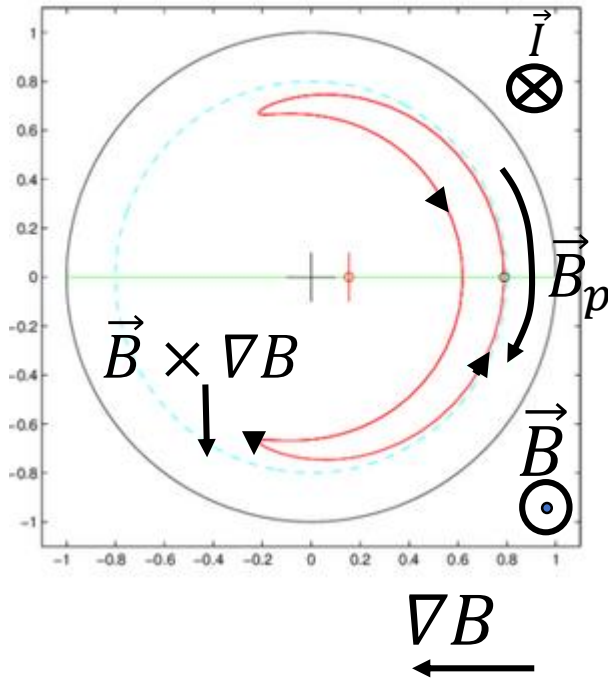
$$\eta_0 \approx \hat{x} \cdot \hat{\phi} = \frac{R_T}{R}$$

Trapped particles injected near boundary



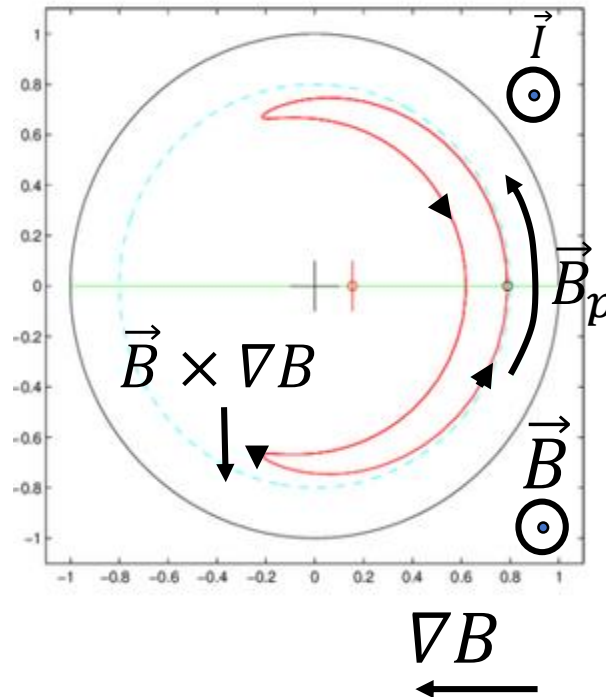
Interlude: in which direction is a trapped ion travelling on its outer leg?

- In which direction does an ion travel on the outer leg of a banana orbit?
- At the banana tip $\vec{v} = \vec{v}_{\parallel} + \vec{v}_d$



On the outer leg the ion is travelling opposite \vec{B} i.e. in the **co-current direction**

26/06/2023



On the outer leg the ion is travelling along \vec{B} i.e. in the **co-current direction**

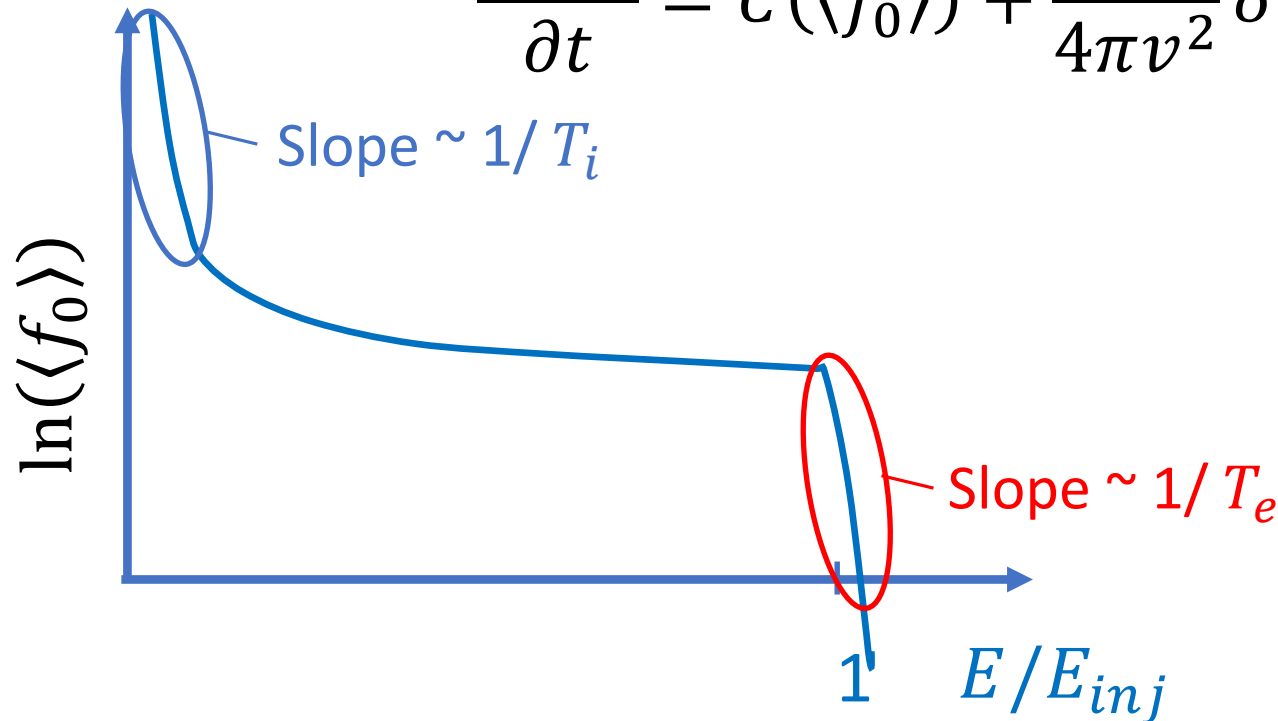
12th ITER International School June 2023

- A trapped ion is always travelling in the co-current direction on the outer leg of its banana orbit
- In order to avoid fast ion losses near the edge, injection must therefore be in the co-current direction

NBI distribution function

- One can show that in the small banana width limit an appropriately defined pitch angle averaged distribution function $\langle f_0 \rangle$, satisfies

$$\frac{\partial \langle f_0 \rangle}{\partial t} = C(\langle f_0 \rangle) + \frac{S_0}{4\pi v^2} \delta(v - v_0)$$



- Full neoclassical calculation of NBI slowing down distribution with TRANSP has been found to be consistent with CTS measurements in ASDEX-Upgrade¹

¹S K Nielsen et al 2015 Plasma Phys. Control. Fusion 57 035009

- For assessing the collisional power transfer to bulk plasma ions and electrons we can use the same approximate formula as for alpha particles with $v_{\alpha 0}$ exchanged for v_{inj}

$$p_e = p_{NBI} \left[1 - \frac{2v_c^2}{3v_{inj}^2} \left\{ + \frac{1}{2} \ln \left[1 - \frac{v_{inj}}{v_c} + \left(\frac{v_0}{v_c} \right)^2 \right] - \ln \left(1 + \frac{v_{inj}}{v_c} \right) + \sqrt{3} \left[\arctan \left(\frac{2v_{inj}}{v_c} - 1 \right) + \frac{\pi}{6} \right] \right\} \right]$$

- For D injection at $E_0 = 1MeV$ into a DT plasma with $n_e = 1 \cdot 10^{20} m^{-3}$:

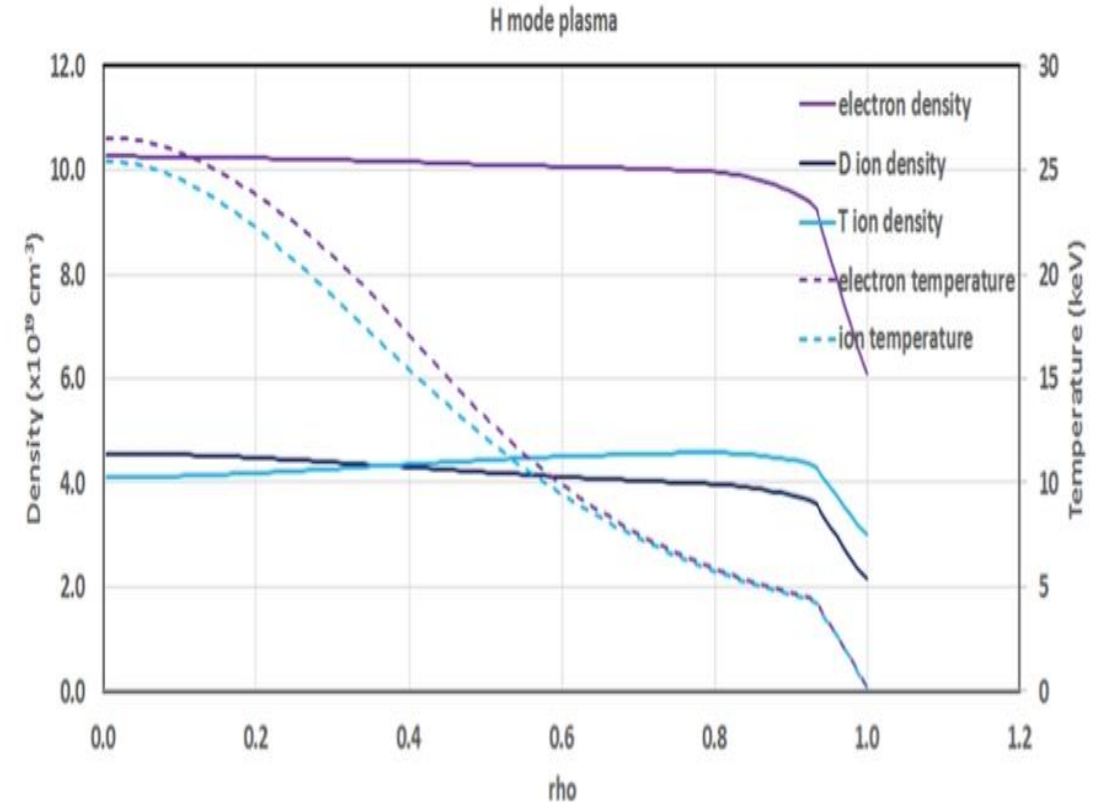
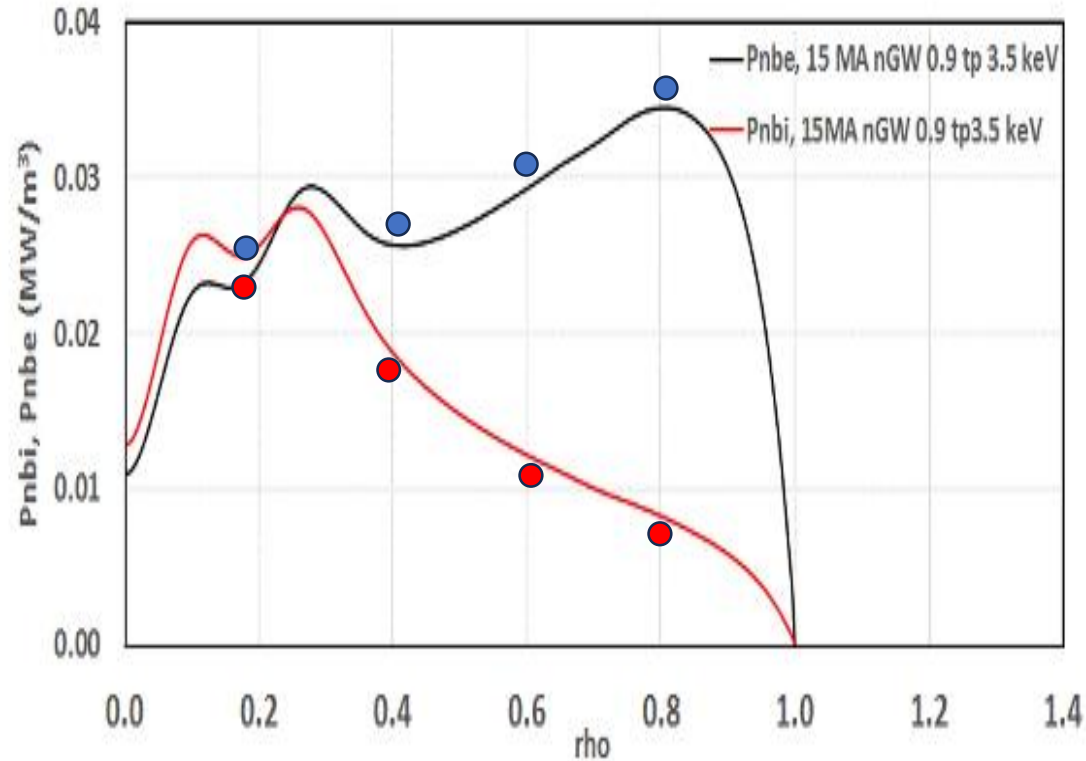
With $T = 25 keV$ ($\frac{1}{2}mv_c^2 \approx 410keV$) we obtain

$$p_e \approx 0.5p_{NBI} \quad \text{and} \quad p_i \approx 0.5 P_{NBI}$$

With $T = 10 keV$ ($\frac{1}{2}mv_c^2 \approx 165keV$) we obtain

$$p_e \approx 0.73p_{NBI} \quad \text{and} \quad p_i \approx 0.27p_{NBI}$$

- Simulated power density transferred to the plasma ions and electrons by 33 MW NBI for a 15 MA/5.3 T Q=10 ITER DT plasma

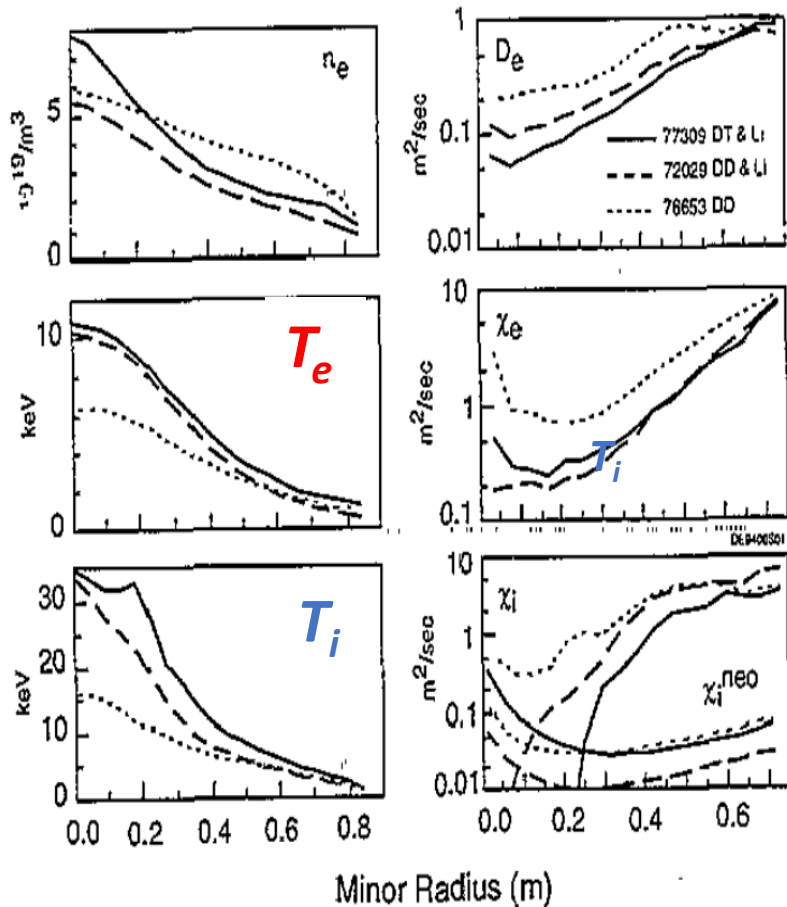


● “Simple” formula

M J Singh et al. New J. Phys. **19** (2017)055004

• Deuterium tritium experiment with high NBI power in TFTR and JET

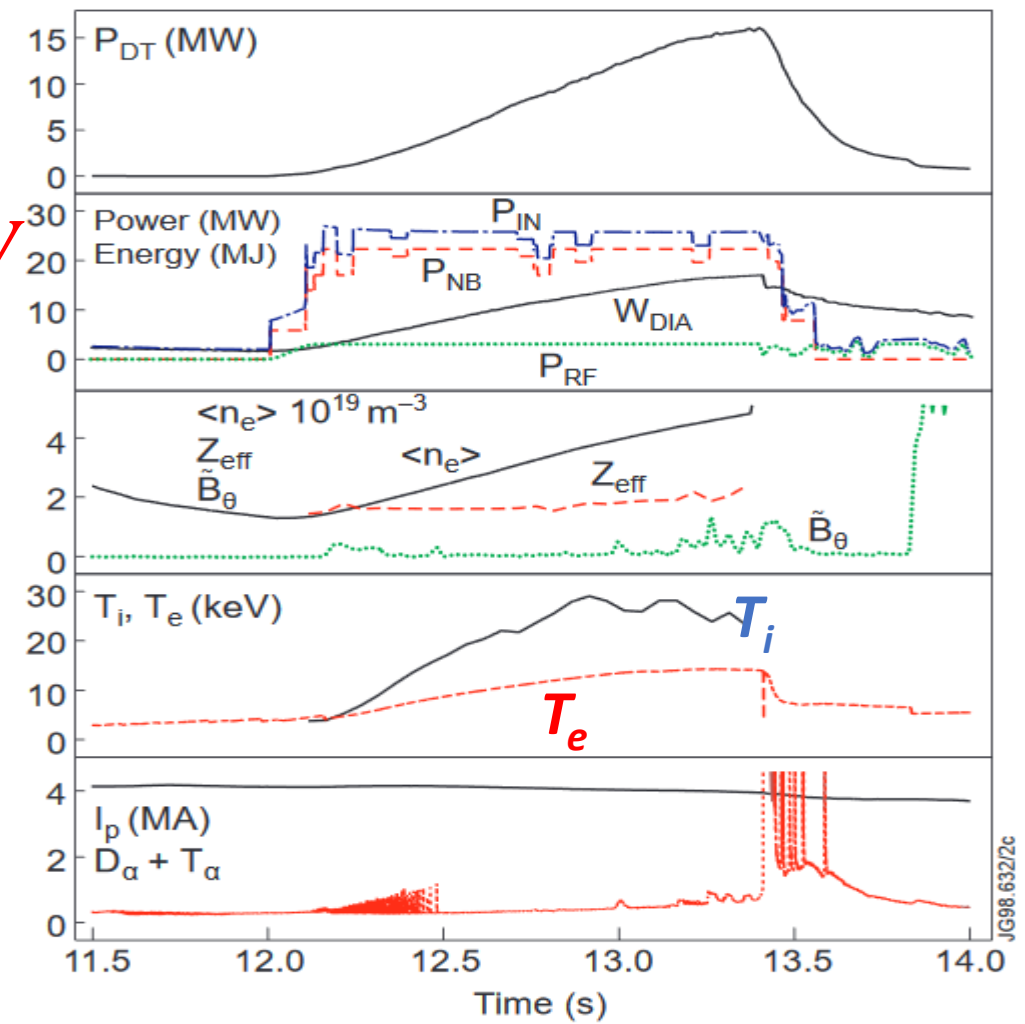
TFTR



$E_{inj} \sim 100 \text{ keV}$
 $\frac{1}{2} m v_c^2 \sim 170 \text{ keV}$

T_i significantly greater than T_e

JET

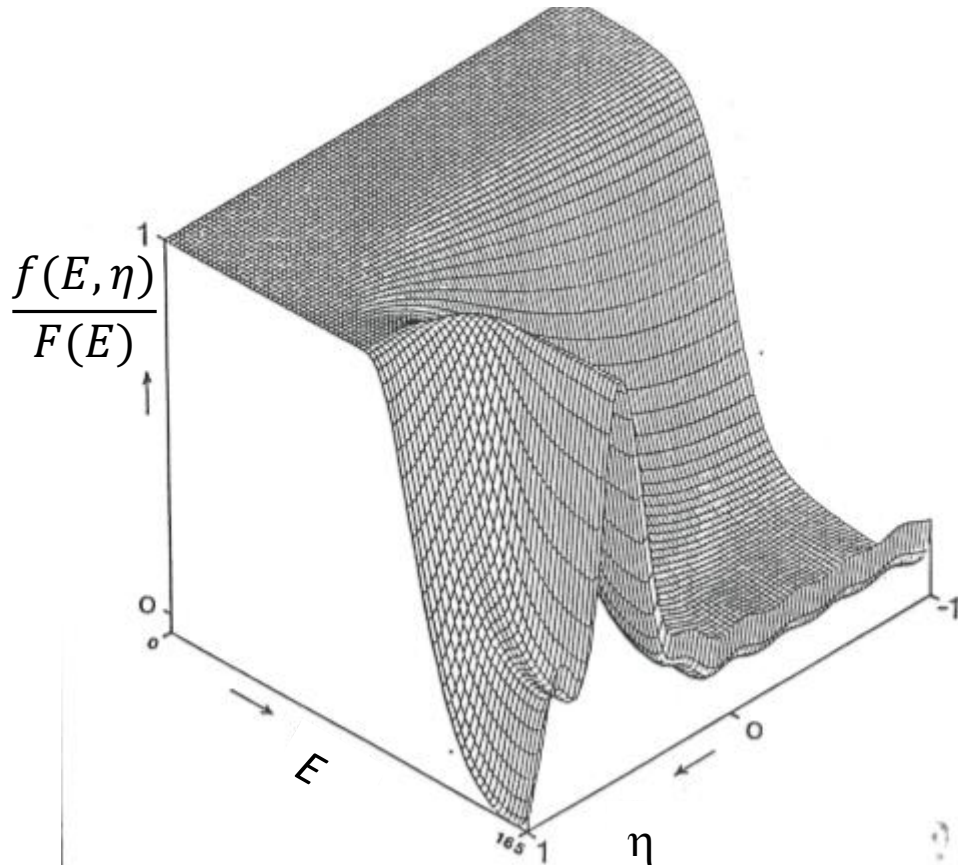


J.D Strachan et al Plasma Phys. Control. Fusion **36** (1994) B3-B15.

M. Keilhacker et al 1999 Nucl. Fusion **39** 209

Neutral beam current drive

- Non-perpendicular NBI obviously give rise to a fast ion current



- To get some insight we consider a source of the type,

$$S = \frac{S_0}{4\pi v_0^2} \delta(v - v_0) \delta(\eta - \eta_0) \delta(\theta - \theta_0)$$

With η_0 close to one and

$$E_{inj} = \frac{1}{2} m v_0^2$$

- We can write the flux surface averaged fast ion current as

$$j_{NBf} = Ze \left\langle \int_0^\infty 2\pi v^2 dv \int_{-1}^1 v \eta f_0 \left(v, \Lambda(\eta, \psi, \theta), P_\varphi(v, \eta, \psi, \theta) \right) d\eta \right\rangle_{\text{Flux surface}}$$

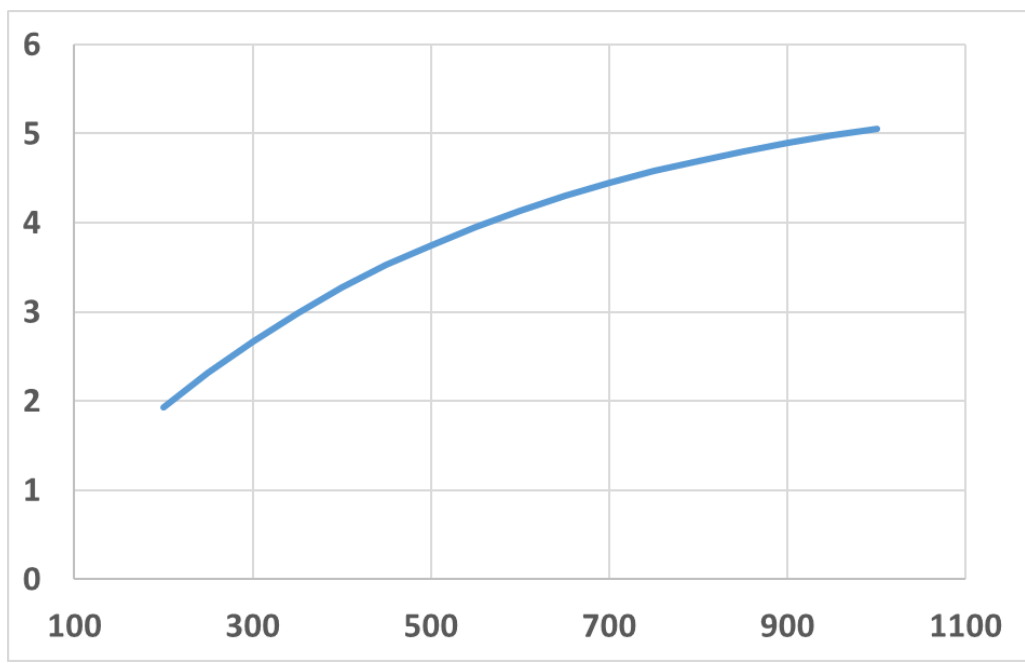
- Neglecting trapped particle effects and FOW effects one finds by expanding f_0 in Legendre polynomials

$$\frac{j_{NBf}}{p_{NBI}} = \frac{Zet_s \eta_0 v_c}{E_{inj}} \left(1 + \frac{v_c^3}{v_0^3} \right)^{\frac{v_\gamma^3}{6v_c^3}} \int_0^{\frac{v_0}{v_c}} \left(\frac{x^3}{x^3 + 1} \right)^{\frac{v_\gamma^3}{6v_c^3} + 1} dx$$

- This represents the upper limit of the fast ion current density (trapping effects will reduce it).

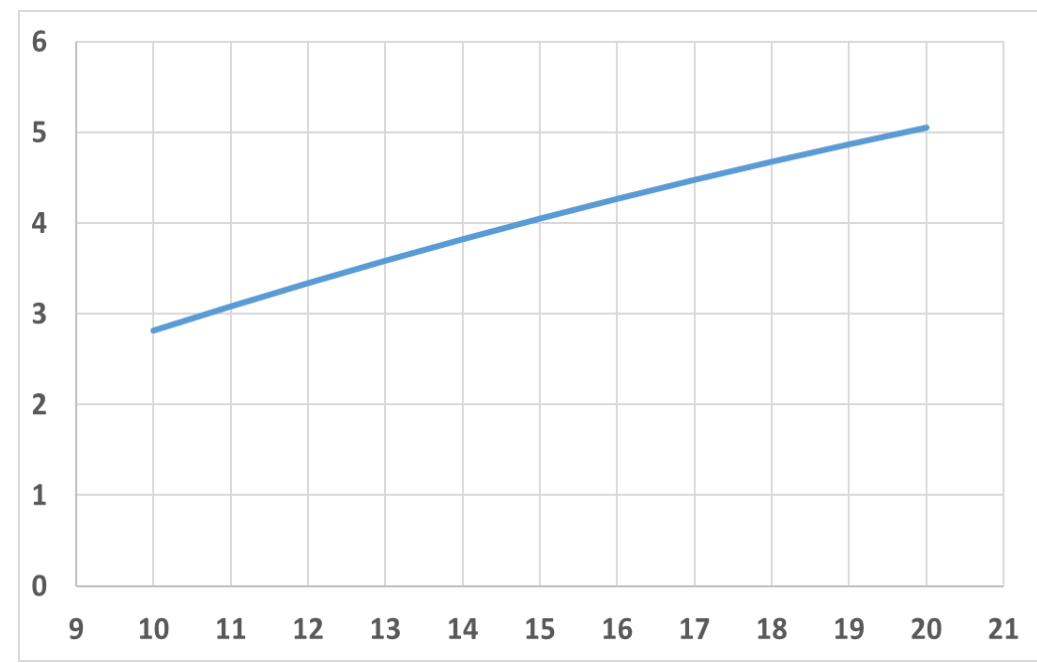
- ITER type plasma, tangential D injection (i.e. $\eta = 1$), $n_e = 1.0 \cdot 10^{20} m^{-3}$

$j_{NBf}/p_{NBI} (Am/W)$



$E_{inj} (keV)$

$j_{NBf}/p_{NBI} (Am/W)$



$T_e (keV)$

Electron back current

- However, the situation is a little more complicated, the fast ions “drags along” electrons, i.e. there is an electron back current

$$j_{NBCD} = j_{NBf} \left[1 - \frac{Z}{Z_{eff}} (1 - G) \right]$$

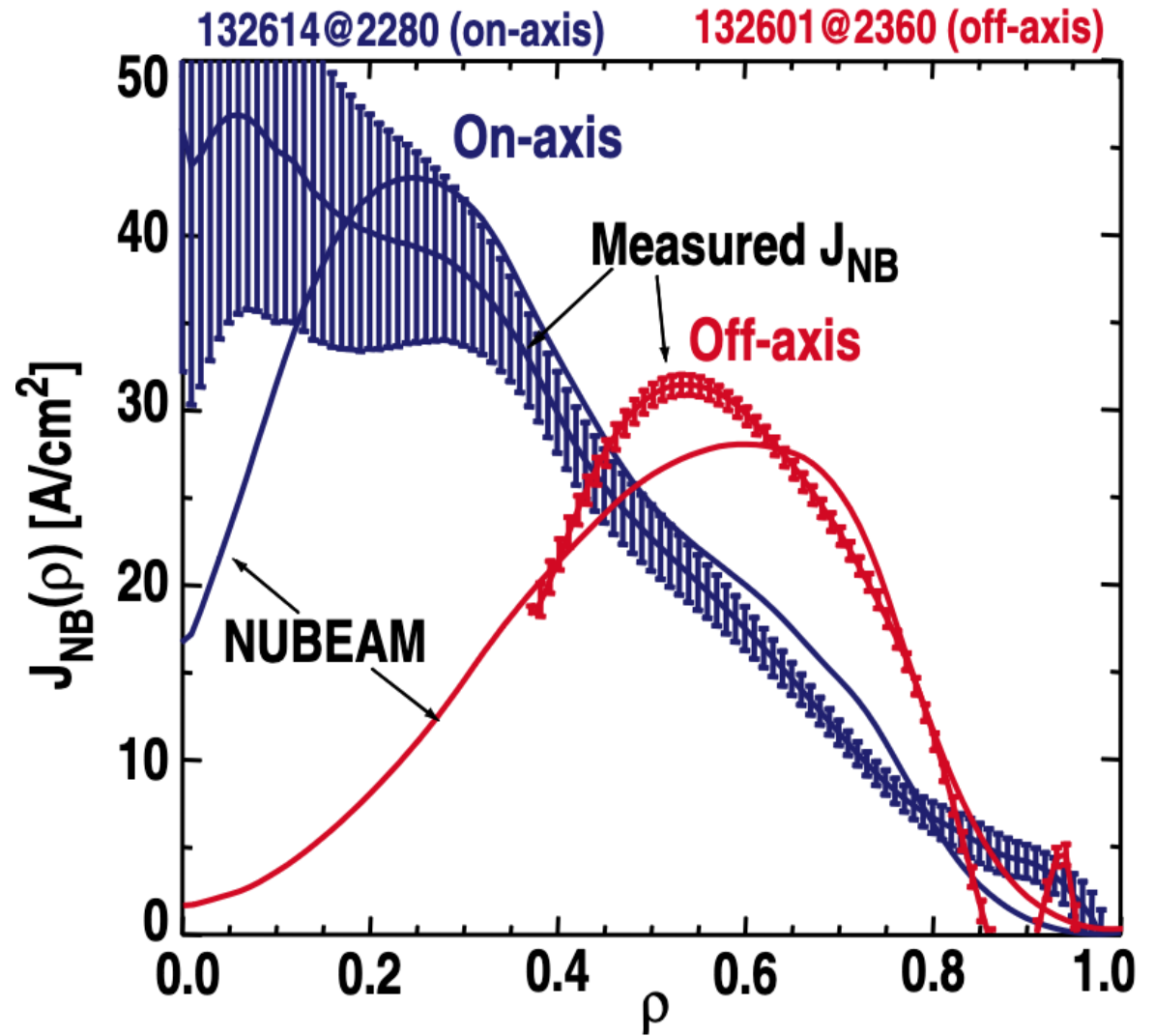
- To calculate the radially dependent term G is complicated*. An approximate expression is $G = 1.46\sqrt{\varepsilon}A(Z_{eff})$; with $A(Z_{eff}) \sim 1-4$ for $1 < Z_{eff} < 4$.
- For $Z_{eff} = 2$, we can see that the total current is about 50% of the fast ion current in a D or DT plasma.

See e.g M. Honda et al, Nuclear Fusion, **52**, 023021 (2012)

Comparison experiment and simulations

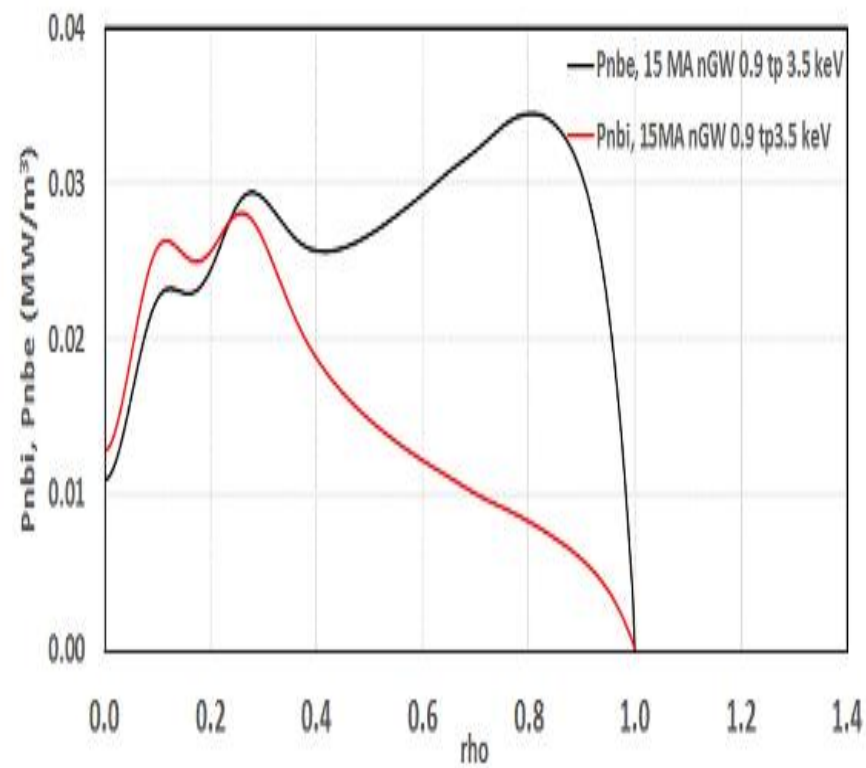
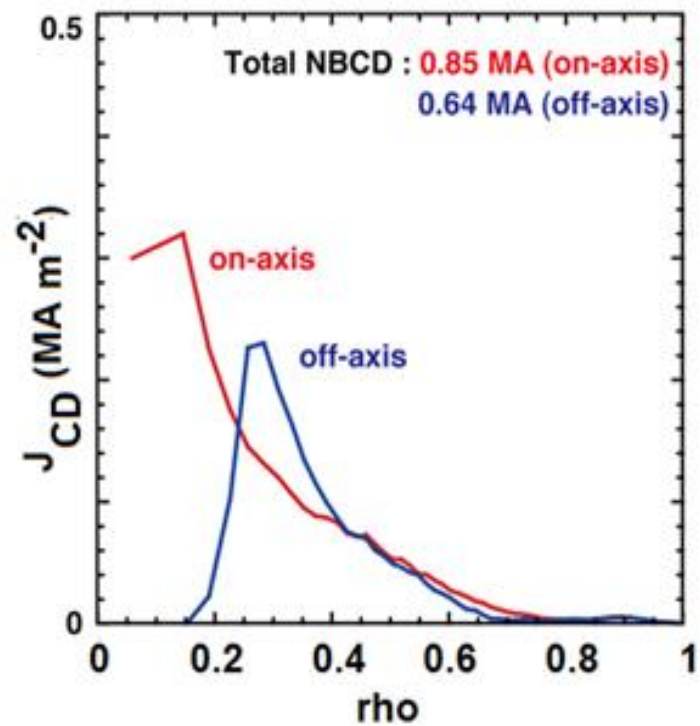
- Experiments on DIII-D with on and of axis NBI.
- Generally good agreement with the orbit following Monte Carlo Code NUBEAM (used in TRANSP).
- Virtually no anomalous effects found D.C. Pace et al. PoP **20**, 056108 (2013)
- In ASDEX-Upgrade the situation is less clearcut and anomalous effects may play a role, see Rittich, D.(2018).

<https://hdl.handle.net/21.11116/0000-0002-8E89-4>



J.M. Park et al Physics of Plasmas **16**, 092508 (2009)

- Simulation for $P_{\text{NBI}} = 33 \text{ MW}$ ITER plasma

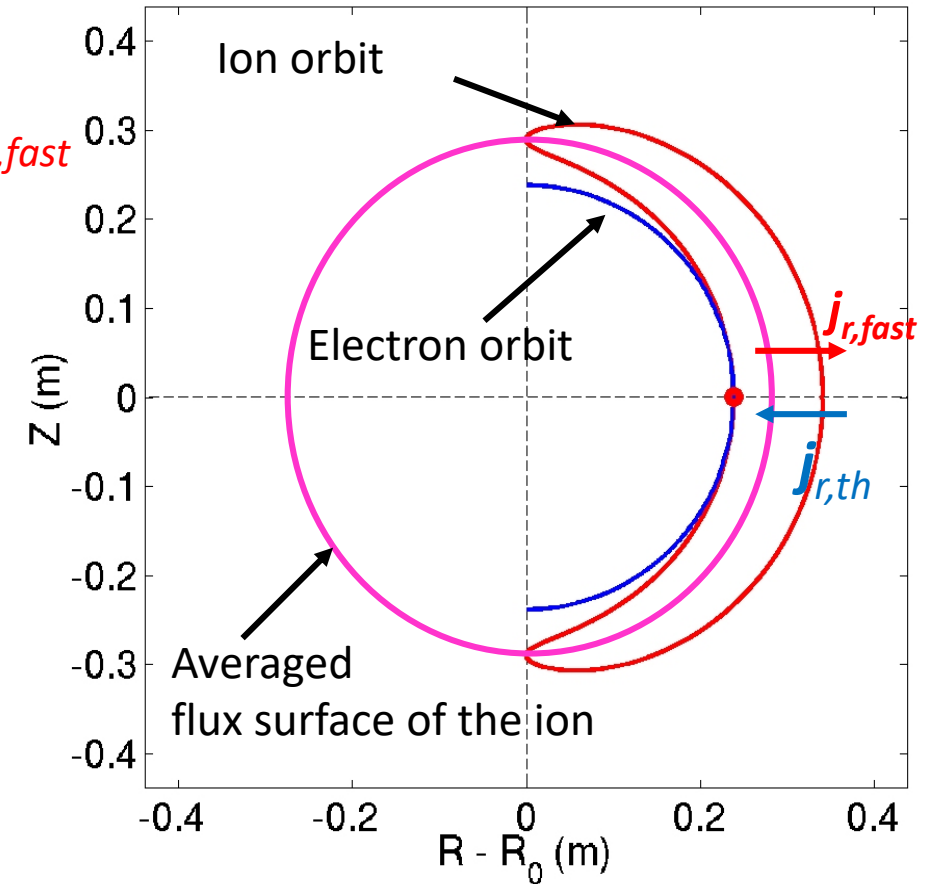
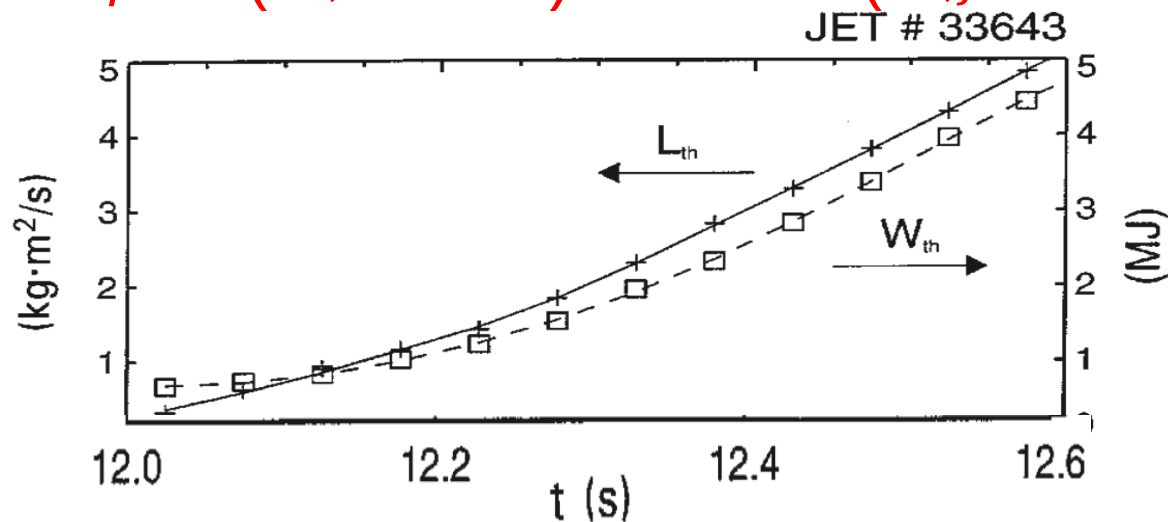


NBI and rotation: momentum transfer

- Passing NBI ions via collisional slowing down/pitch angle scattering.
- NBI ions born on trapped orbits: transfer their momentum during one bounce time

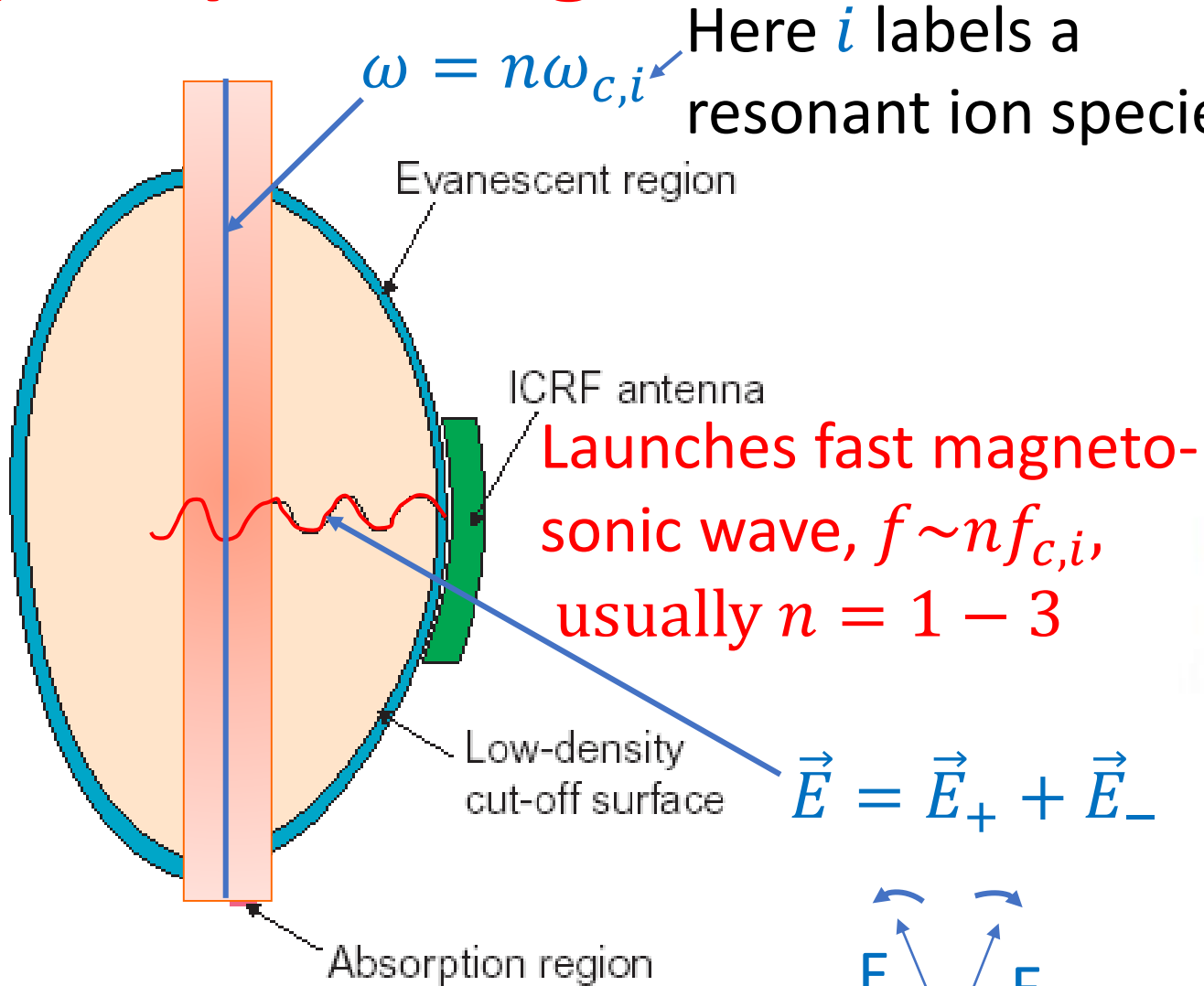
- Electron and ion orbits have different width $\rightarrow j_{r,fast}$
- Quasi neutrality \rightarrow thermal radial current
 $\vec{J}_{r,th} = -\vec{J}_{r,fast} \rightarrow$

$$T_\varphi = (\vec{J}_{r,th} \times \vec{B}) \cdot \hat{\varphi} = -(\vec{J}_{r,fast} \times \vec{B}) \cdot \hat{\varphi}$$

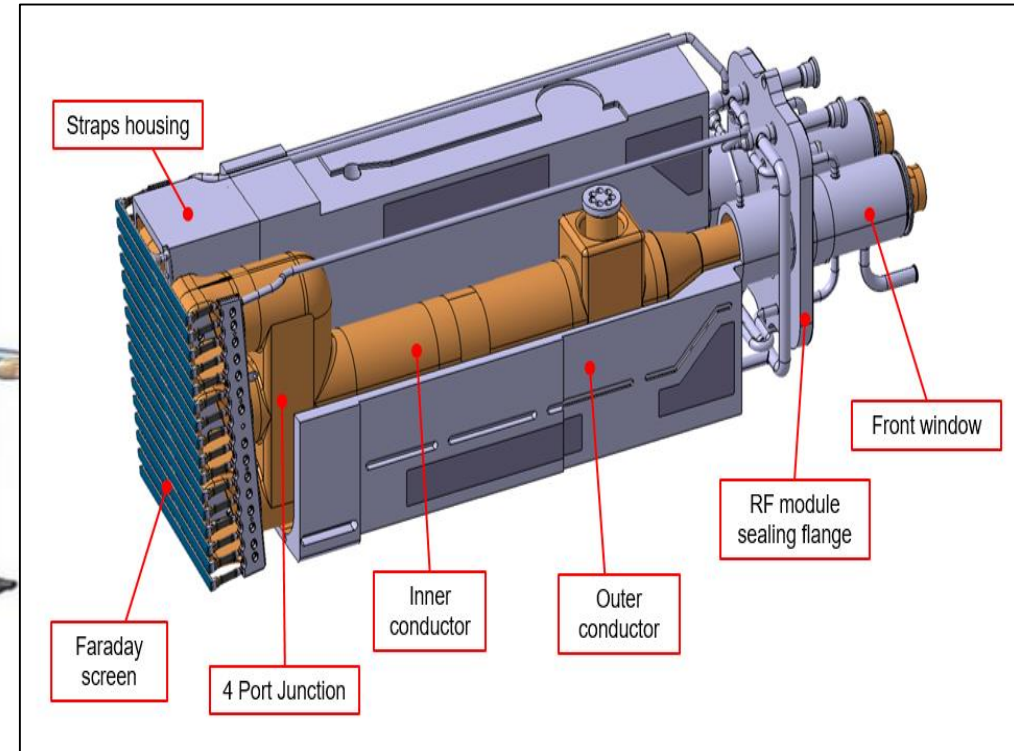


Ion Cyclotron Resonance Frequency (ICRF) heating

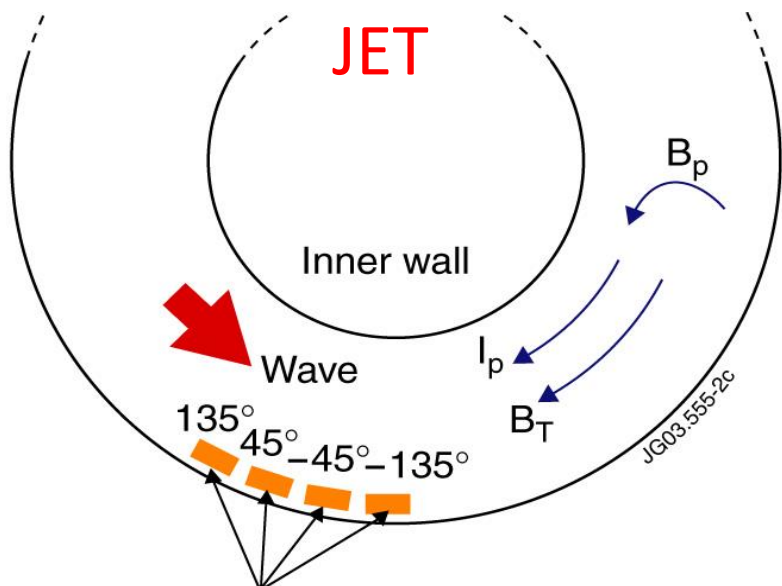
Here i labels a resonant ion species



ITER ICRF antenna

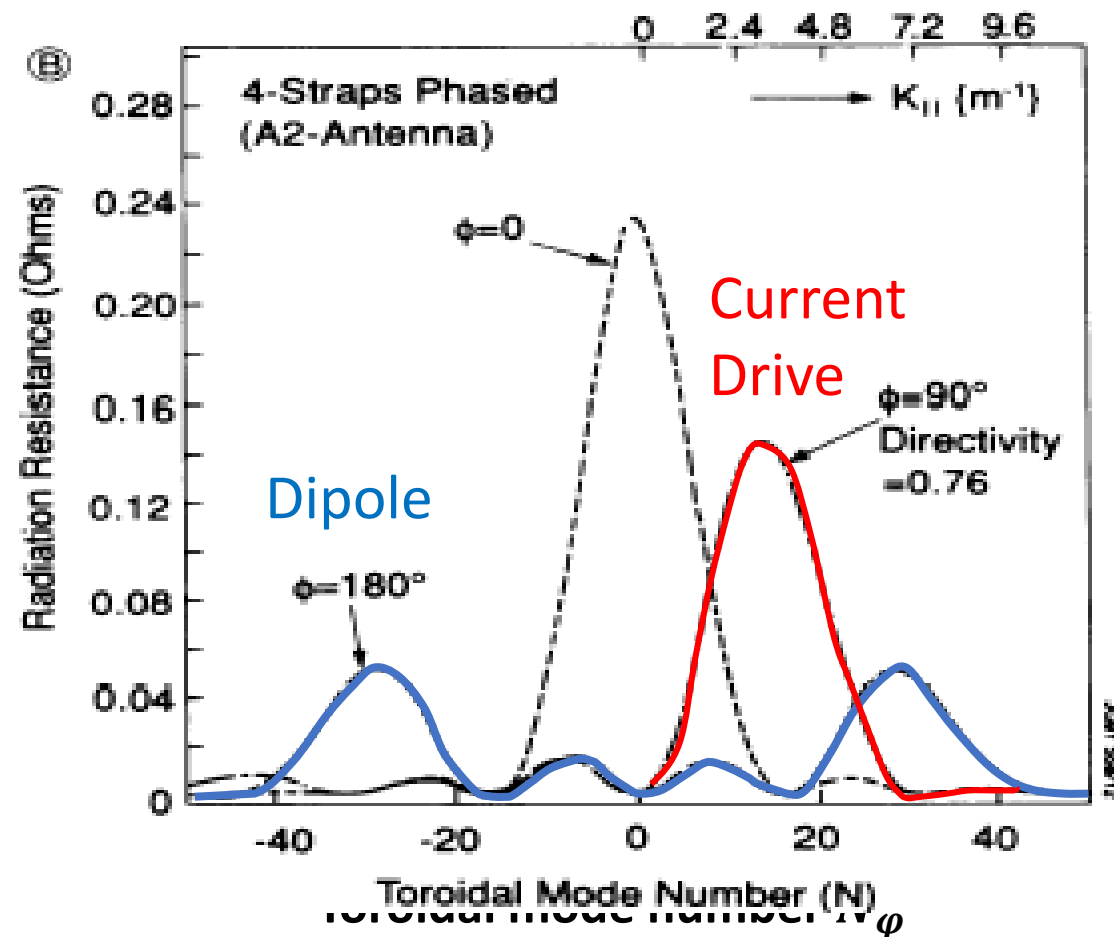
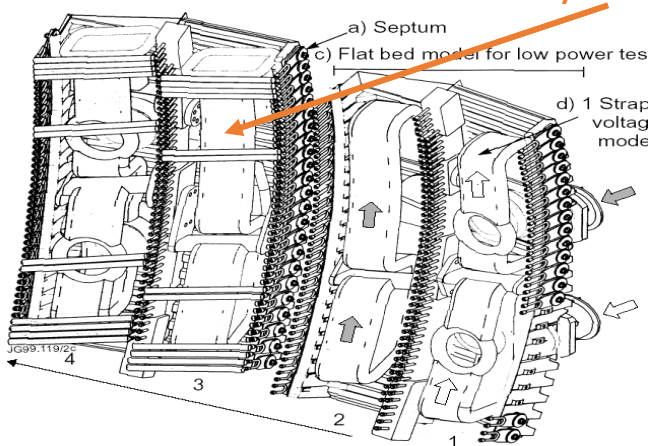


ICRF antenna spectra



Current straps for -90° Phased Waves

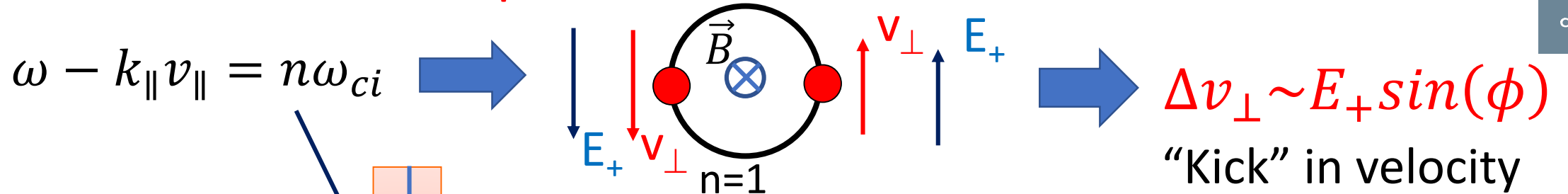
Faraday screen bars



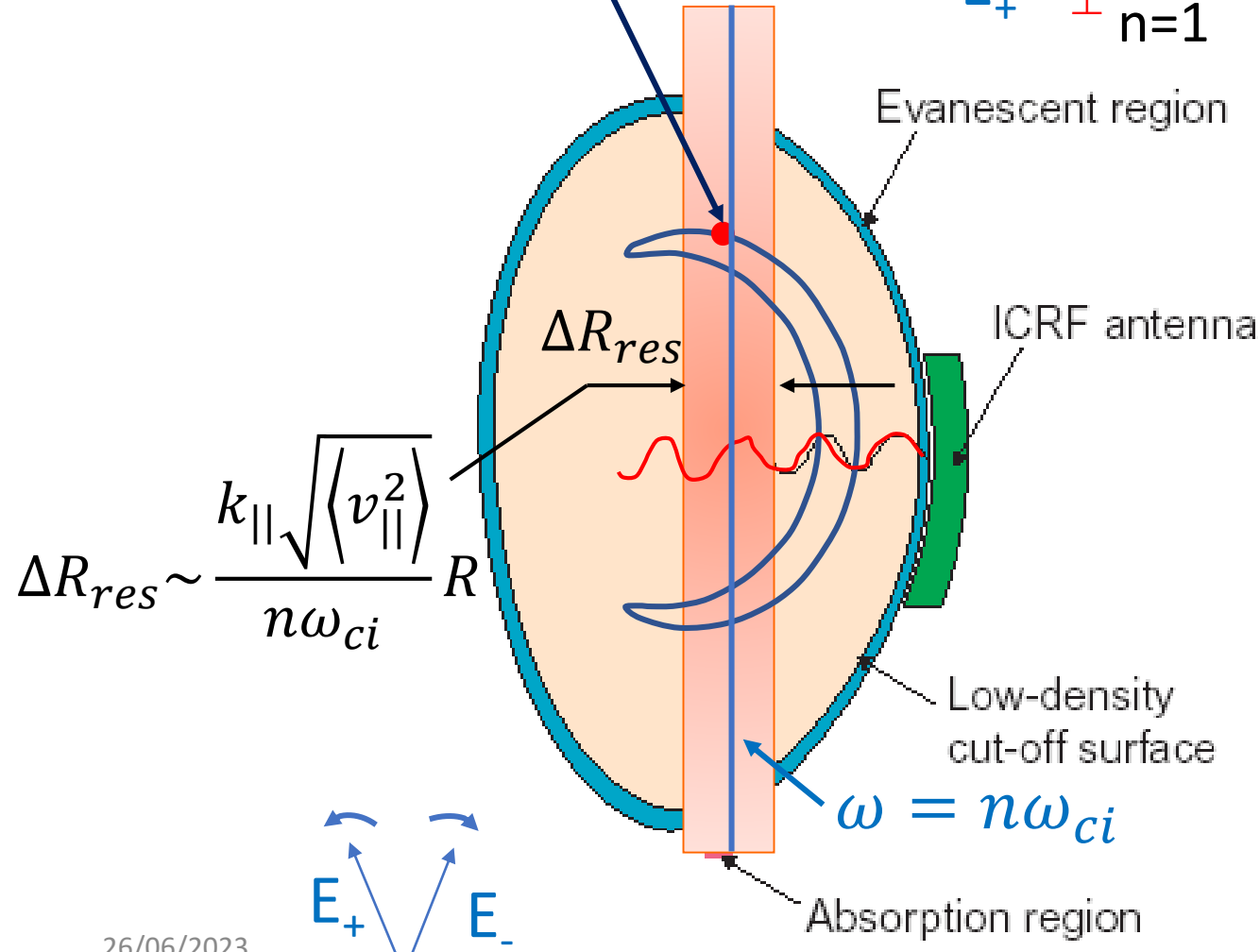
$$\vec{E} = \vec{E} e^{iN_\phi \phi}$$

$$k_{||} \approx \frac{N_\phi}{R} \text{ for } N_\phi \gg 1$$

Wave particle interaction



- ϕ is the phase between the cyclotron motion and the wave phase;
- E_{+} is the left hand polarised component, rotating the same direction of the ions, of the wave electric field.
- We will see later that also the E_{-} component can give rise to a “kick” in velocity, but it is an FLR effect.



Effective ICRF heating – a random walk process

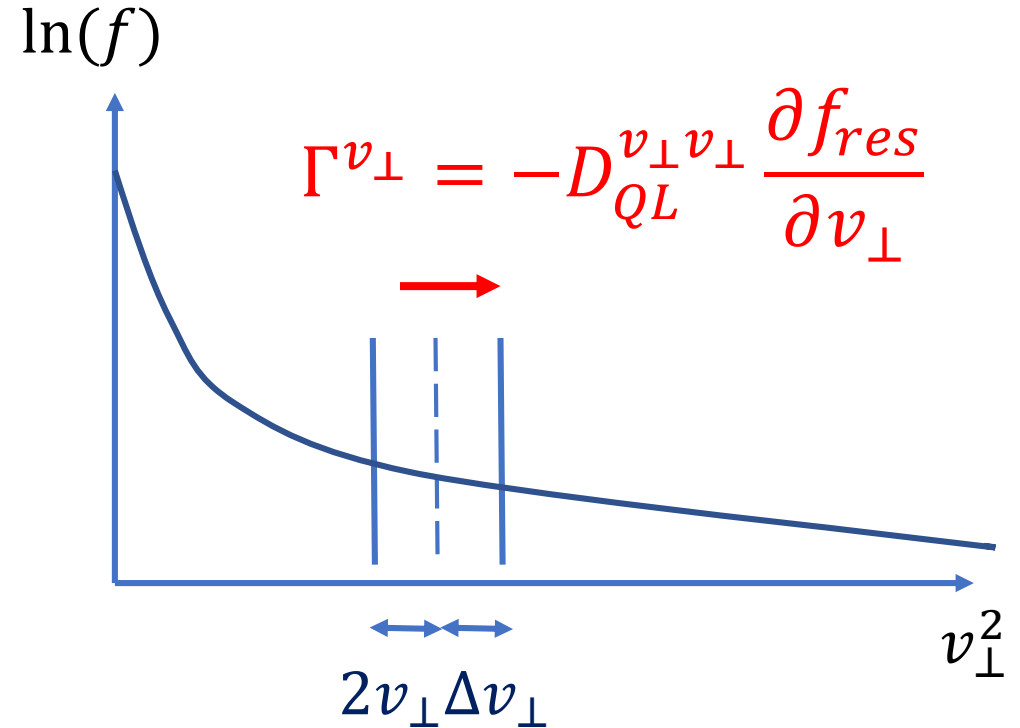
- Effective ICRF heating requires that ϕ is randomised between successive “kicks”^{1,2,3},
- ϕ randomised between successive “kicks” \rightarrow random walk process characterised by:

$$D_{QL}^{v_{\perp}v_{\perp}} \sim \frac{\langle (\Delta v_{\perp})^2 \rangle |_{\phi}}{2\tau_b}$$

- For n=1 and lowest order in $k_{\perp}v_{\perp}/\omega_{ci}$

$$D_{QL}^{v_{\perp}v_{\perp}} \sim |E_+|^2$$

- Thus, ICRF heating is essentially a diffusive process in velocity space.

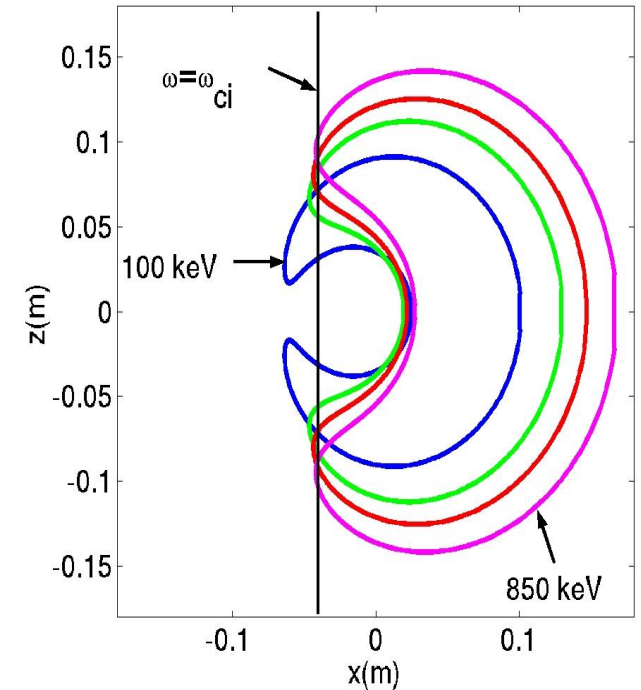
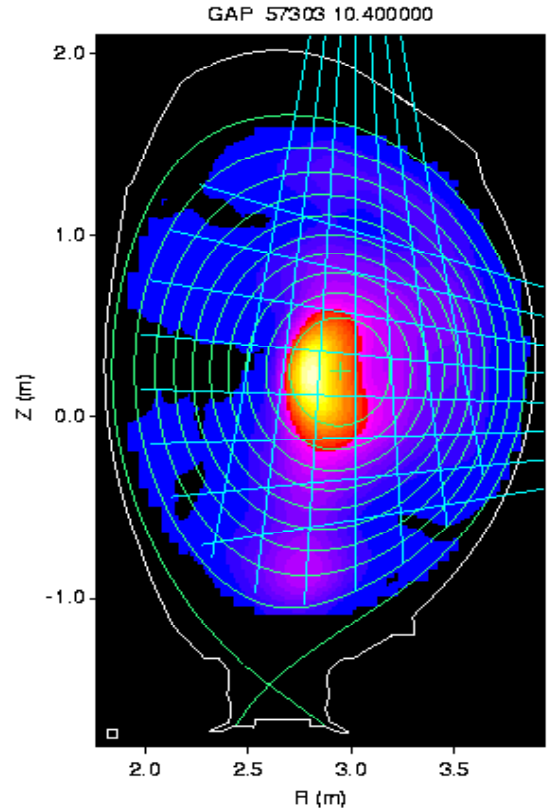
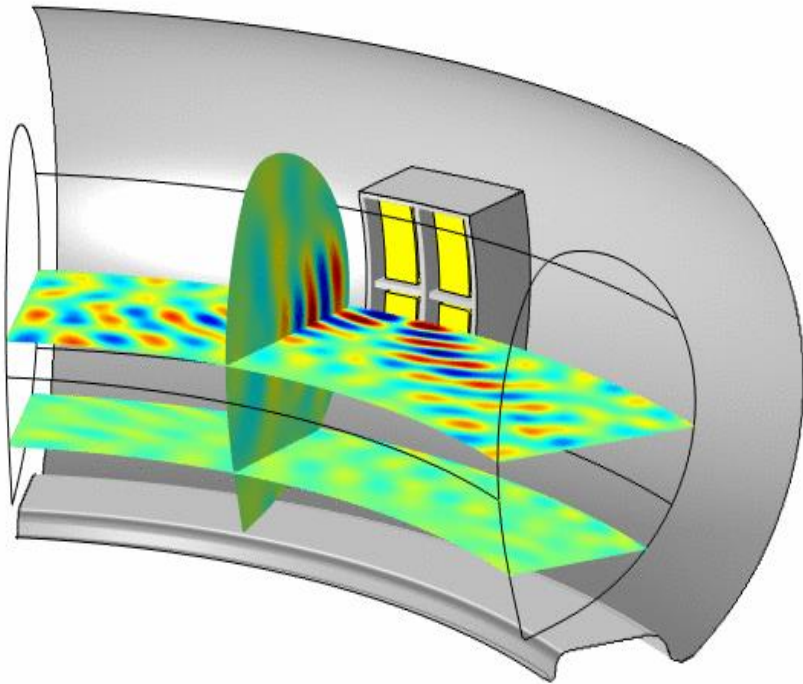


¹T.H. Stix “Waves in Plasmas” AIP 1992

²P. Helander, M. Lisak Phys. Fluids **B 4** (7), 1927

³V. Bergaud et al. Phys. Plasmas, **8**, 2001, 139

ICRF physics modelling in a nutshell



$$\nabla \times \nabla \times \vec{E} = \hat{\epsilon}(f_{0,1}, \dots, f_{0,n}) \cdot \vec{E} + i\omega\mu_0 \vec{J}_{ext}$$

$$\frac{\partial f_{0,n}}{\partial t} = \langle C(f_{0,n}) \rangle + \langle Q(f_{0,n}, \vec{E}) \rangle$$

ICRF schemes

Cold plasma dispersion for single ion plasma :

$$\frac{E_+}{E_-} = \frac{\frac{\omega_{ci}}{\omega + \omega_{ci}} - N_{\parallel}^2}{\frac{\omega_{ci}}{\omega - \omega_{ci}} \omega + N_{\parallel}^2}$$



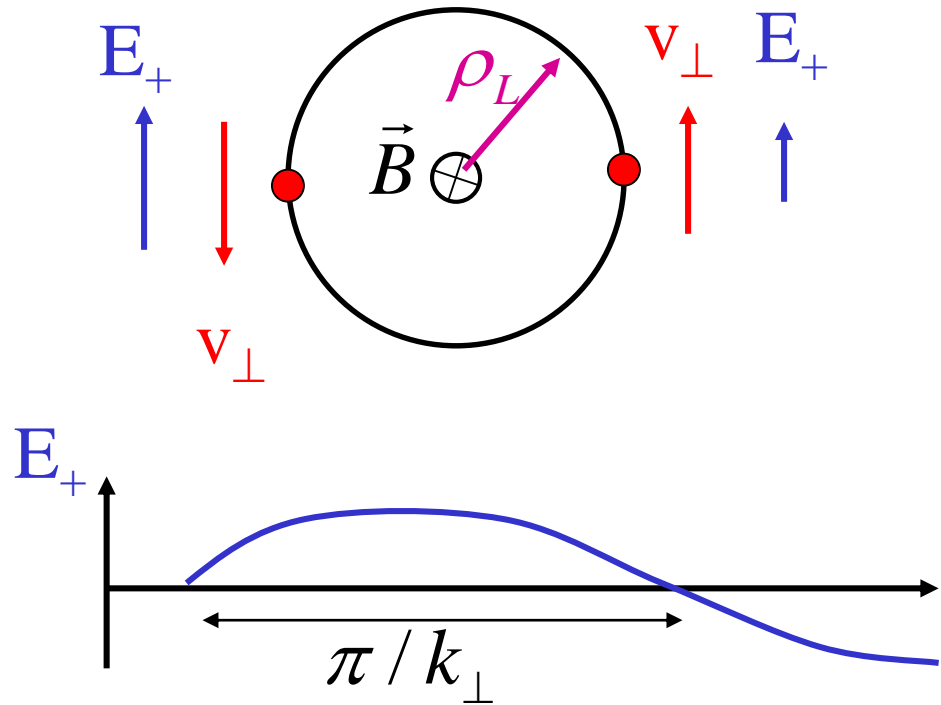
$\omega \sim \omega_{ci}$ does not work for a majority ion species

□ There are a few options:

- **Minority heating:** introduce a minority ion species that is resonant with the waves, which has higher cyclotron frequency than the majority species, (H)D, (³He)D, (D)T, the ion-ion hybrid layer that is introduced ensures a significant E_+ at the $\omega = \omega_{c,min}$, typically $n_{min}/n_{maj} \sim 1 - 10\%$. **High power and minority absorption → energetic ions**
- **Higher harmonic heating:** $\omega = n\omega_{c,maj}$, $n \geq 2$, the ITER ICRF system is designed for $\omega = 2\omega_{c,T}$ at the full magnetic field. **Absorption is an FLR effect → energetic ions**
- **Three ion scheme:** in a plasma with two majority species introduce a third, minority, species with $\omega = \omega_{c,min}$ where E_+/E is max. **Ye. O. Kazakov et al Nature Physics 13, (2017)**

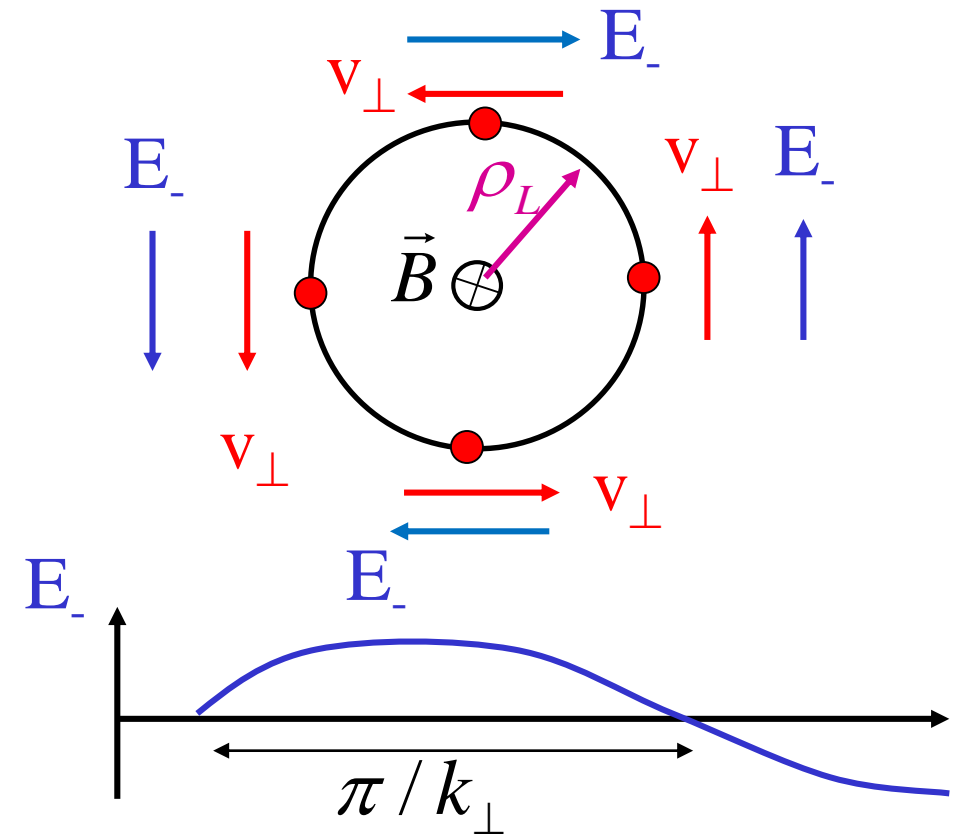
“Kick” mechanisms for $n > 1$ and E_-

$n = 2; E_+$



$$\Delta v_{\perp} \sim E_+ \left(\frac{k_{\perp} v_{\perp}}{\omega} \right) \sin(\phi)$$

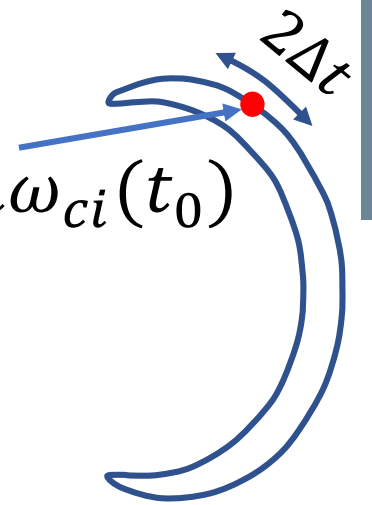
$n = 1; E_-$



$$\Delta v_{\perp} \sim E_- \left(\frac{k_{\perp} v_{\perp}}{\omega} \right)^2 \sin(\phi)$$

The kick and stationary phase

$$\omega - k_{\parallel} v_{\parallel} = n\omega_{ci}(t_0)$$

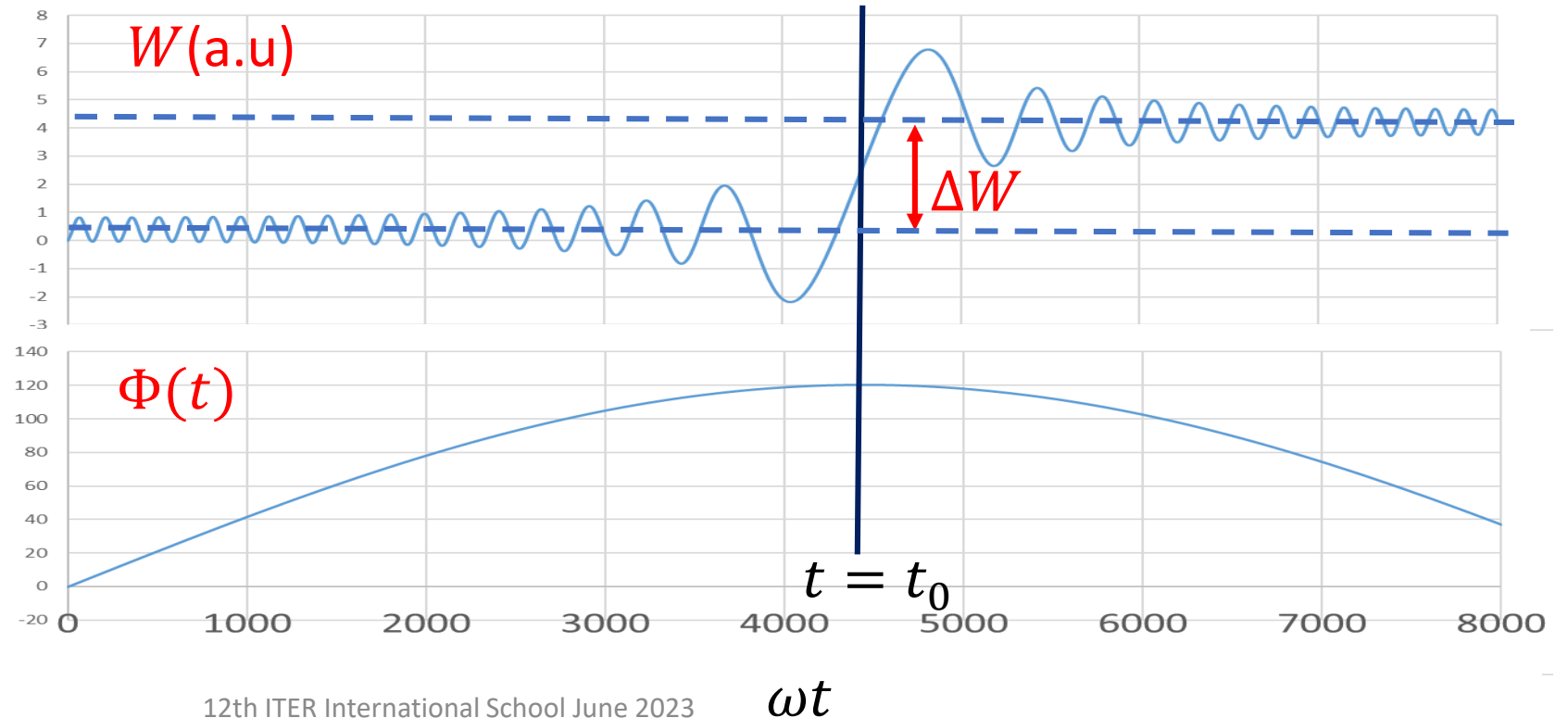


- The change in energy of a particle is given by

$$\Delta W = \int_{t-\Delta t}^{t+\Delta t} Ze\vec{E} \cdot \vec{v} dt \approx \int_{t_0-\Delta t}^{t_0+\Delta t} Ze\{E_{\perp} v_{\perp} \cos[\Phi(t) + \phi] + \dots\} dt$$

ICRF ↓

$$\frac{d\Phi}{dt} = (\omega - k_{\parallel} v_{\parallel} - n\omega_{ci})$$



Stationary phase approximation

- The averaged square of the “energy kick” formulated in terms of a random walk diffusion coefficient can be calculated using the stationary phase approximation,

$$D_{N,n,\omega,Res}^{WW} = \frac{\langle (\Delta W)^2 \rangle |_{\phi}}{2\tau_b} \approx \frac{\pi(Ze)^2 v_{\perp R}^2}{2\tau_b |n\dot{\omega}_{cR}|} \left| E_+ J_{n-1} \left(\frac{k_{\perp} v_{\perp R}}{\omega_{cR}} \right) + E_- J_{n+1} \left(\frac{k_{\perp} v_{\perp R}}{\omega_{cR}} \right) \right|^2$$

- FLR effects are represented by the Bessel functions, and we will see later that they can be very important.

Relations for wave particle interaction via quantum mech.

$$\Delta W = \hbar\omega$$

$$m\Delta v_{||} = k_{||}\hbar$$

$$\Delta P_{\varphi} = Rk_{\varphi}\hbar = N_{\varphi}\hbar$$

Resonance condition

$$\Delta W_{||} = mv_{||}\Delta v_{||} = \frac{k_{||}v_{||}}{\omega} \Delta W$$

$$\Delta W_{\perp} = \left(1 - \frac{k_{||}v_{||}}{\omega}\right) \Delta W = \frac{n\omega_c}{\omega} \Delta W$$

$$\Delta\Lambda = \Delta \frac{W_{\perp}B_0}{WB} = \left(\frac{\Delta W_{\perp}}{W} - \frac{W_{\perp}}{W^2} \Delta W\right) \frac{B_0}{B} = \left(\frac{n\omega_c}{\omega W} - \frac{W_{\perp}}{W^2}\right) \frac{B_0}{B} \Delta W$$



$$\Delta\Lambda = \frac{n\omega_{c0} - \Lambda\omega}{\omega W} \Delta W$$

$$\Delta P_{\varphi} = \frac{N_{\varphi}}{\omega} \Delta W$$

The orbit averaged Quasi-linear operator $\langle Q(f_0) \rangle$

- Let's denote $\vec{I} = (W, \Lambda, P_\varphi)$.
- We assume that the the energy kicks between resonances are randomised \rightarrow random walk diffusion coefficient,

$$D_{QL}^{ij} = \sum_{n,N,\omega} \sum_{Res.} \frac{\langle \Delta I^i \Delta I^j \rangle |_\phi}{2\tau_b} = \sum_{n,N,\omega} \sum_{Res} D_{n,N,\omega,Res}^{WW} \frac{\partial I^i}{\partial W} \frac{\partial I^j}{\partial W};$$

$$\frac{\partial \Lambda}{\partial W} = \frac{n\omega_{c0} - \Lambda\omega}{\omega W};$$

$$\frac{\partial P_\varphi}{\partial W} = \frac{N_\varphi}{\omega}$$

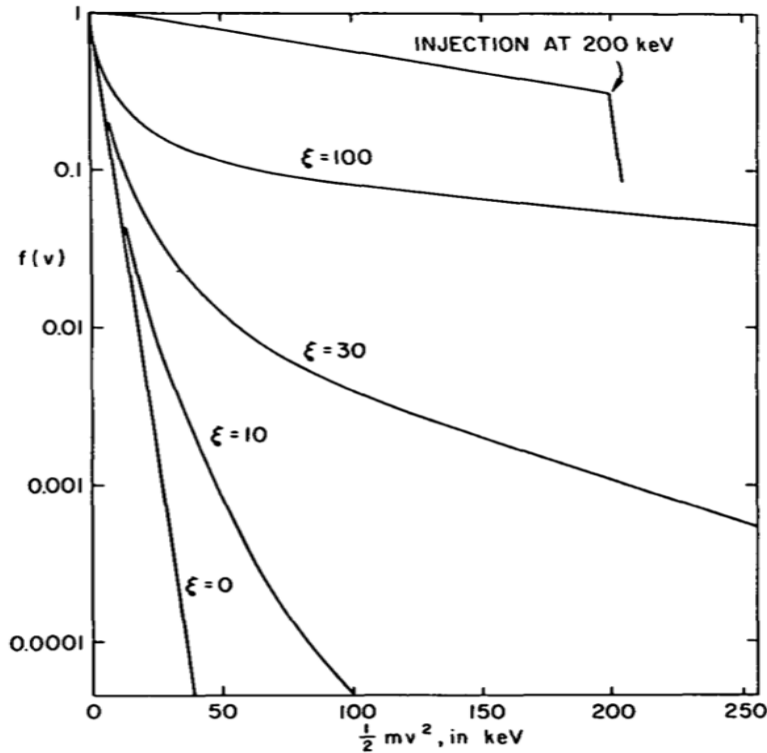
- $\langle Q(f_0) \rangle$ for the wave particle interaction then takes the form:

$$\langle Q(f_0) \rangle = \frac{1}{\sqrt{g}} \frac{\partial}{\partial I^i} \left(\sqrt{g} D_{QL}^{ij} \frac{\partial f_0}{\partial I^j} \right)$$

Properties of ICRF heated distributions

- The collisions are much stronger at low energies than at high \rightarrow development of a non-Maxwellian tail on the distribution.

- $\Delta\Lambda = \frac{n\omega_{c0} - \Lambda\omega}{\omega W} \Delta W \rightarrow \Lambda \xrightarrow{\Delta W > 0} \frac{n\omega_{c0}}{\omega}$; ion with t.p. at resonance, has $\Lambda = \frac{n\omega_{c0}}{\omega}$

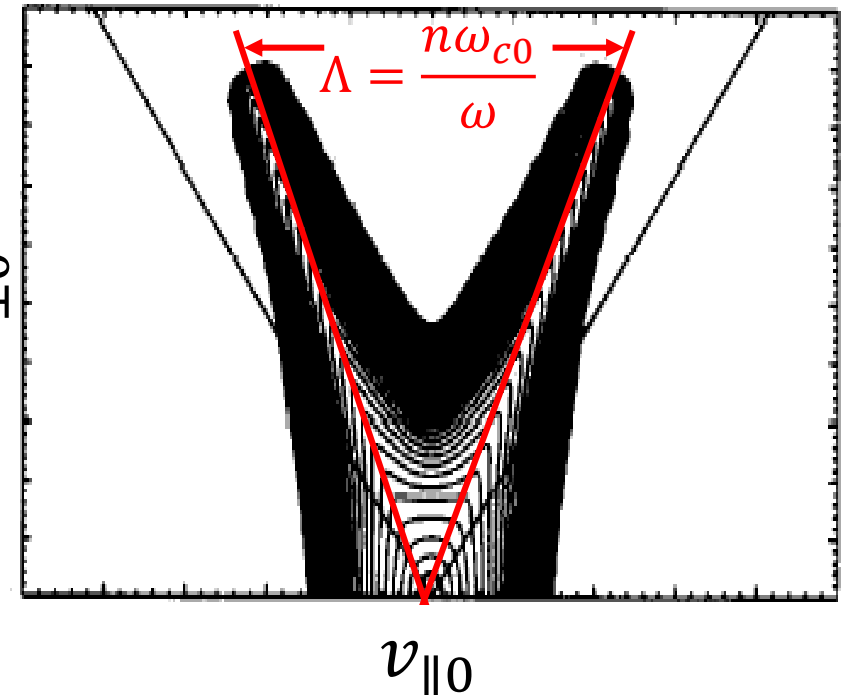


Minority heating

$$\xi kT_e = \frac{p_{ICRF} t_s}{3nm}$$

Stix tail temperature:

$$kT_{tail} \approx kT_e (1 + \xi)$$



Approximate scaling of fast ion energy content

- During high power minority heating a large fraction of the absorbing ions are in the tail of the distribution.
- Retaining only slowing down on electrons, neglecting orbit width \rightarrow

$$0 = \int_r^{r+\Delta r} \frac{1}{2} m v^2 \left[\langle Q(f_0) \rangle + \frac{1}{\sqrt{g}} \frac{\partial}{\partial v} \left(\sqrt{g} \frac{v}{t_s} f_0 \right) \right] \sqrt{g} d^3 I \quad \rightarrow \quad p_{ICRF} - \frac{2W_{fast}}{t_s} = 0$$

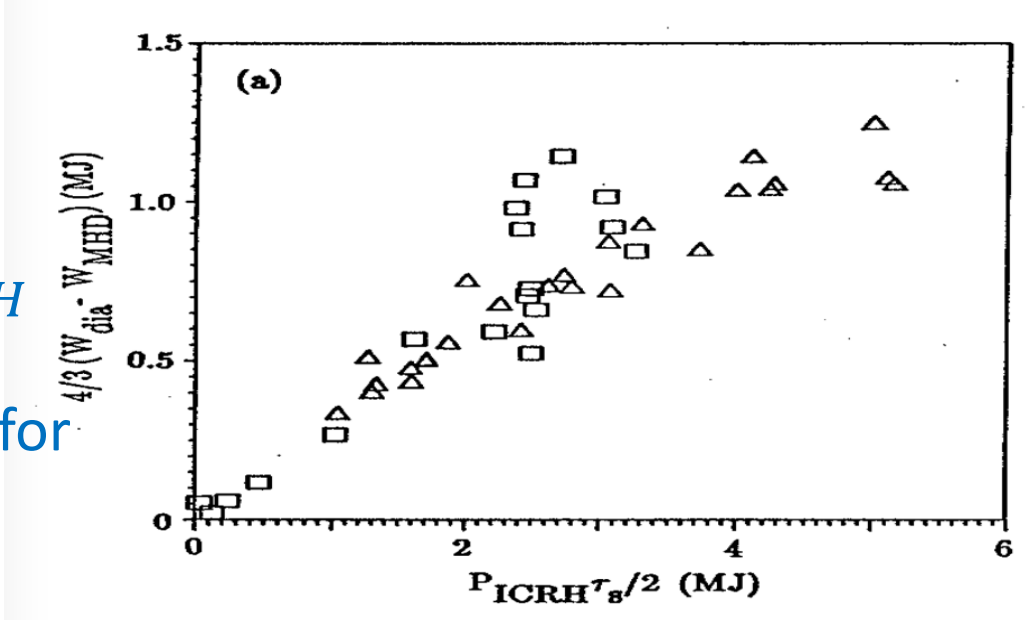
$$W_{fast} \approx \frac{p_{ICRF} t_s}{2}$$

$$t_s \sim \frac{A}{Z^2}$$

$$T_{tail, He-3} < T_{tail, H}$$

$$kT_{tail} \approx \frac{p_{ICRF} t_s}{3nm}$$

Stix tail temperature for $T_{tail} \gg T_e$

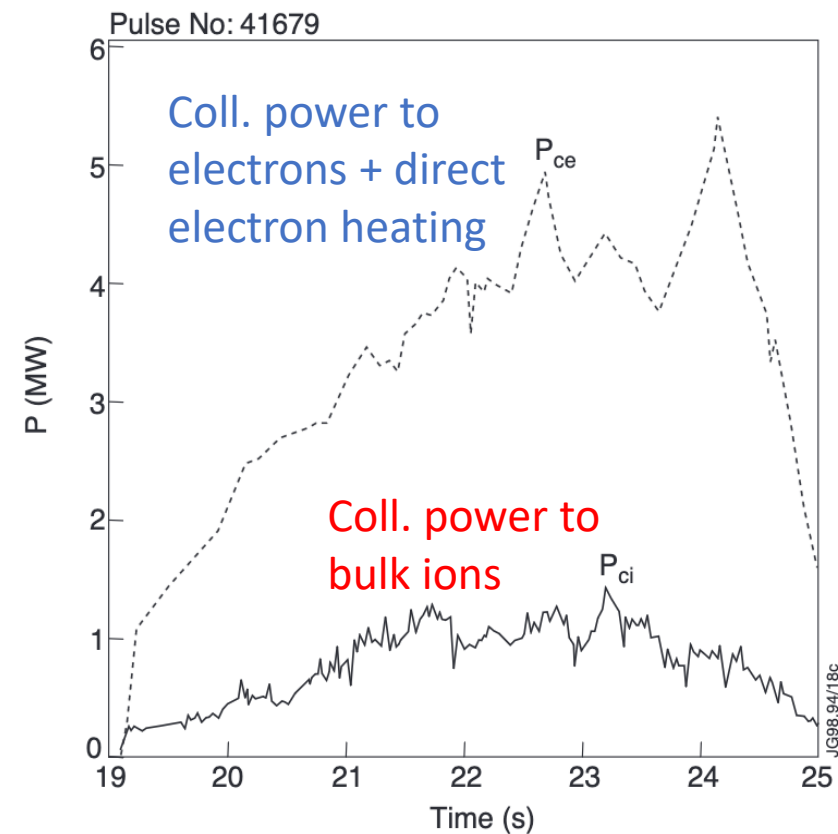
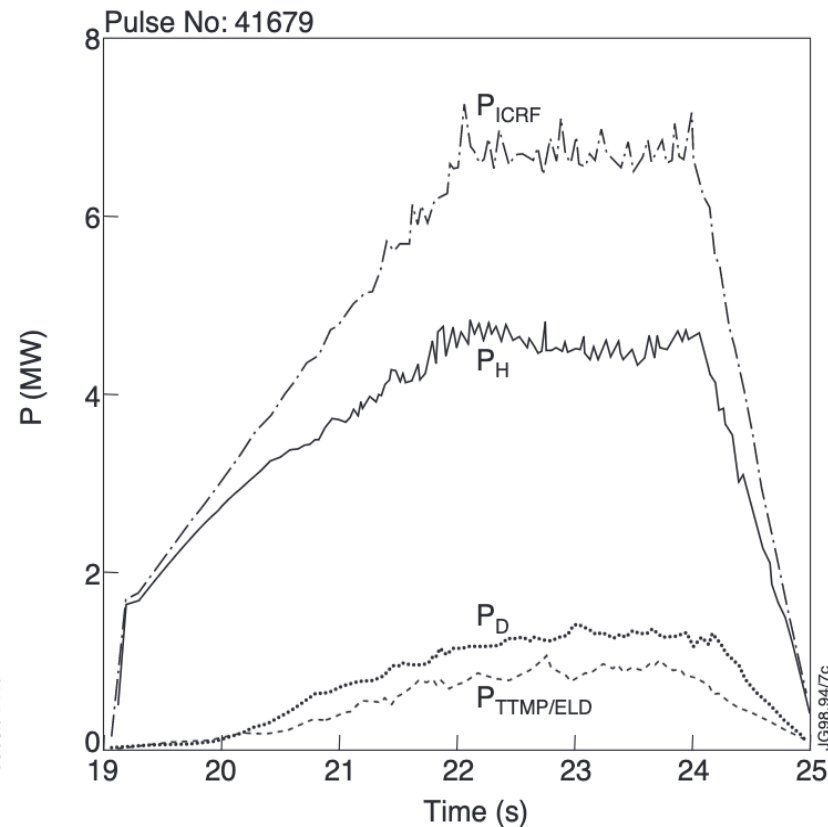
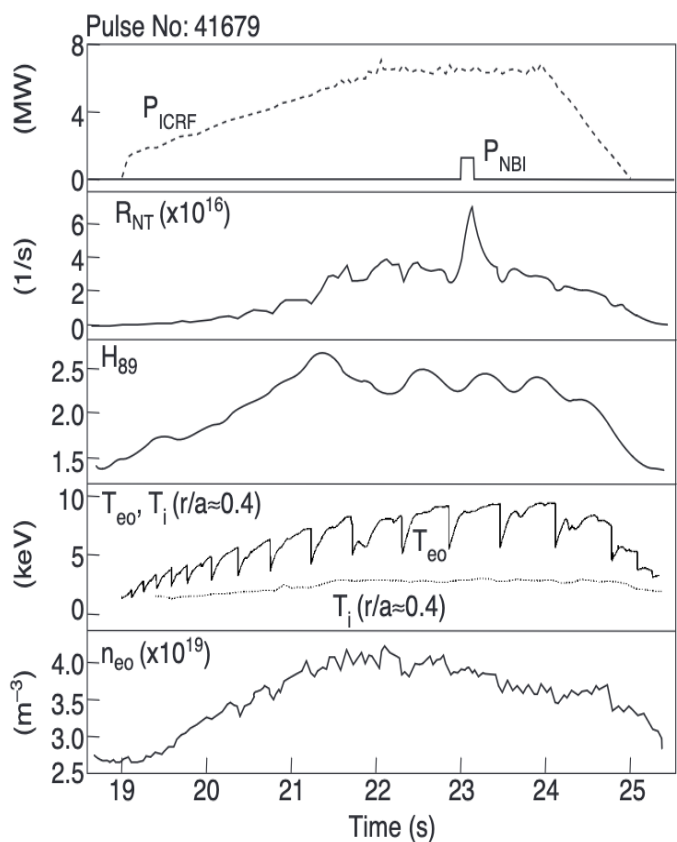


T.H Stix Nuclear Fusion **15**, 737 (1975)

JET team, presented by P.R Thomas (Proc. 12th Int. Conf. Nice, 1988), Vol. 1, IAEA, Vienna (1989) 247.

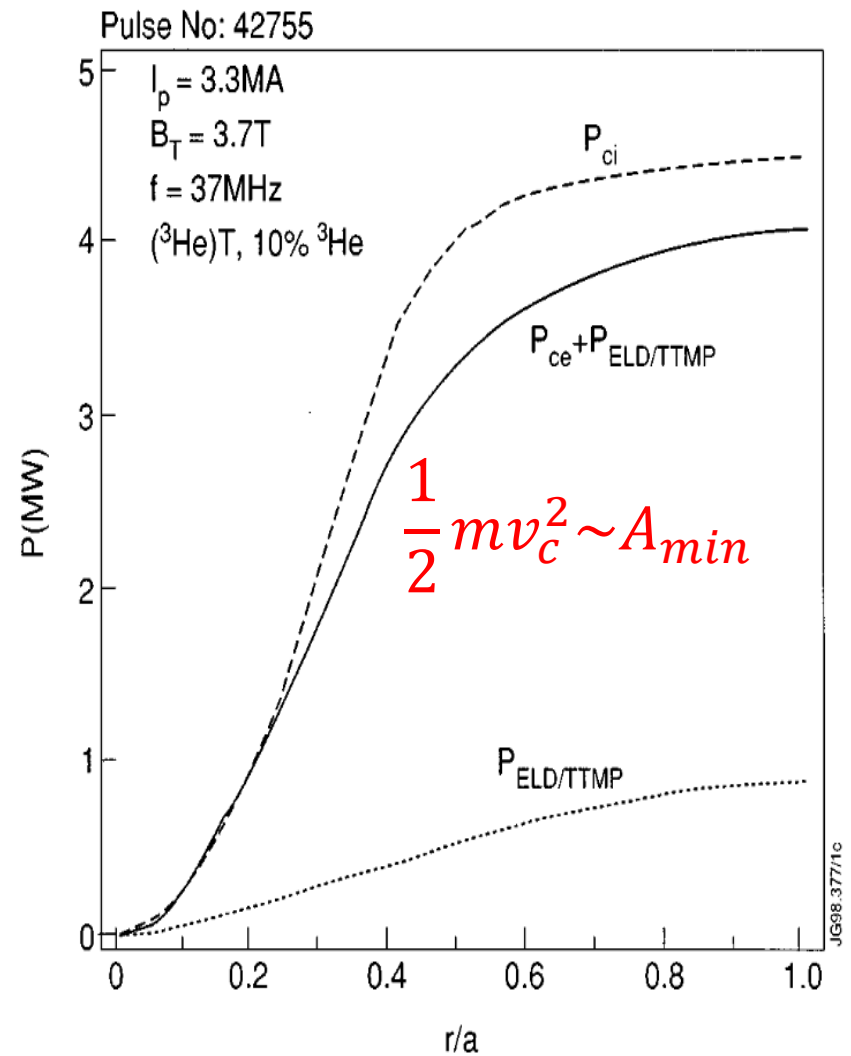
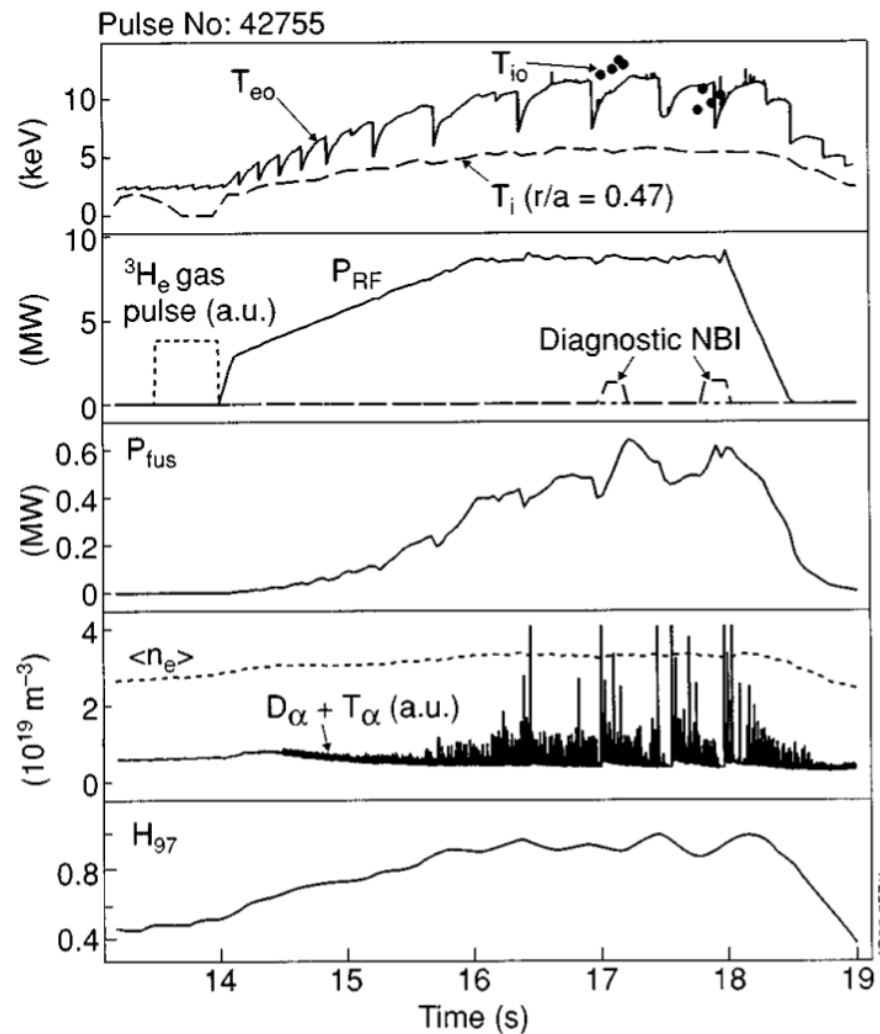
ICRF often heats bulk electrons more than ions

- PION (simplified self consistent wave prop. and FP code) simulation of H minority ICRF in JET DT plasma with $n_H/(n_D + n_T + n_H) \approx 3\%$ \rightarrow
- Simulation of neutron rate agreed well with experiment.



Dominant bulk ion heating with ICRF is possible

- (³He)DT can produce dominant bulk ion heating; example from JET 1997¹
- The three ion scheme with absorption on an heavy impurity is also viable²



¹D.F.H. Start et al 1999 Nucl. Fusion **39** 321; V. Bergeaud et al 2000 Nucl. Fusion **40** 35

²Ye. O. Kazakov et al AIP Conference Proceedings 1689, 030008 (2015)

The distribution and ICRF power deposition

1) Doppler broadening of the resonance: Ex. (³He)D, $f = 30 \text{ MHz}$, $N_\phi = 25$,
 $T_e = 6 \text{ keV}$, $Z_{eff} = 1.5$, \rightarrow
 $T_{\parallel,tail} = 60 \text{ keV}$; $\rightarrow \Delta R_{res} \sim 0.3 \text{ m}$

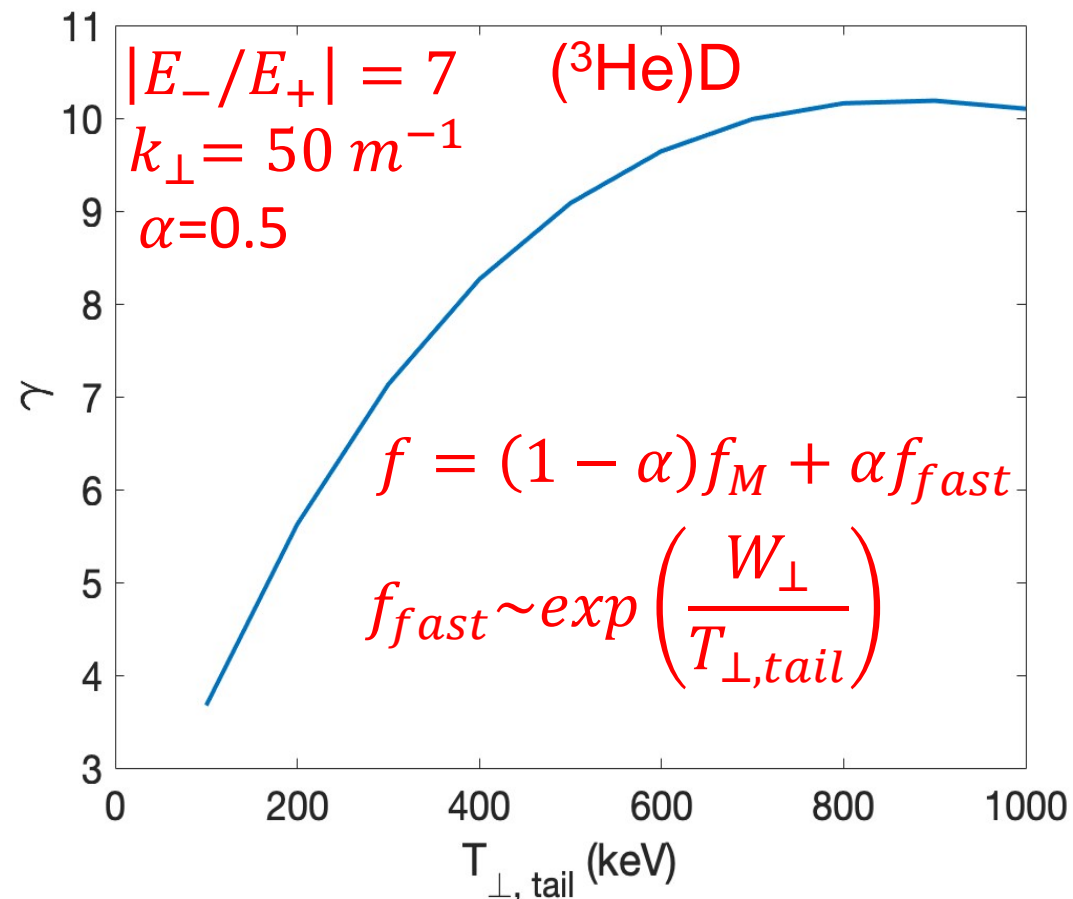
- $\Delta R_{res} \sim R k_{\parallel} \sqrt{2kT_{\parallel,tail}/m}/\omega$
- Stix¹ suggested for $T_{\perp,tail} \gg T_{\parallel,tail}$:

$$T_{\parallel,tail} \sim mv_\gamma^2/8$$

2) FLR effects.

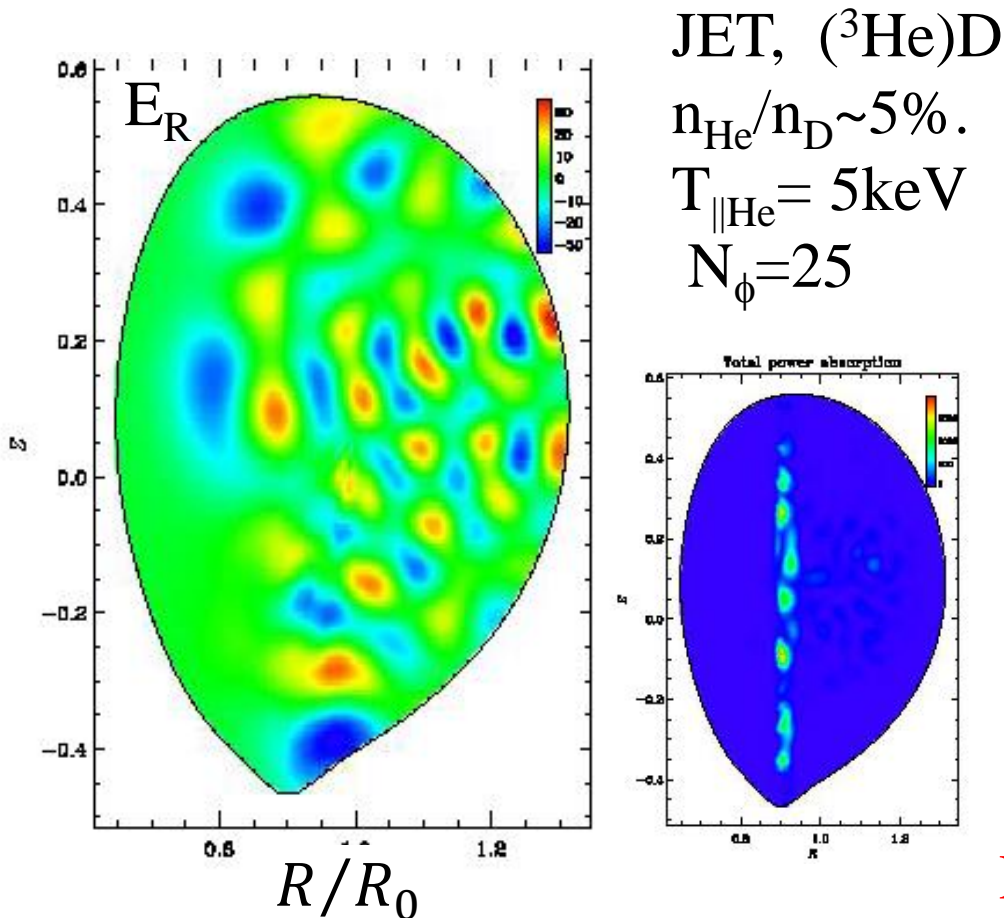
- Wave propagation absorption strength must be equal to that of Fokker-Planck;
- Enhancement over thermal absorption

$$\gamma = \frac{p(f)}{p(f_M)} = \frac{\int mv_\perp^2 D_{QL}^{v_\perp v_\perp} \frac{\partial f}{\partial v_\perp} dv_\perp}{\int mv_\perp^2 D_{QL}^{v_\perp v_\perp} \frac{\partial f_M}{\partial v_\perp} dv_\perp}$$

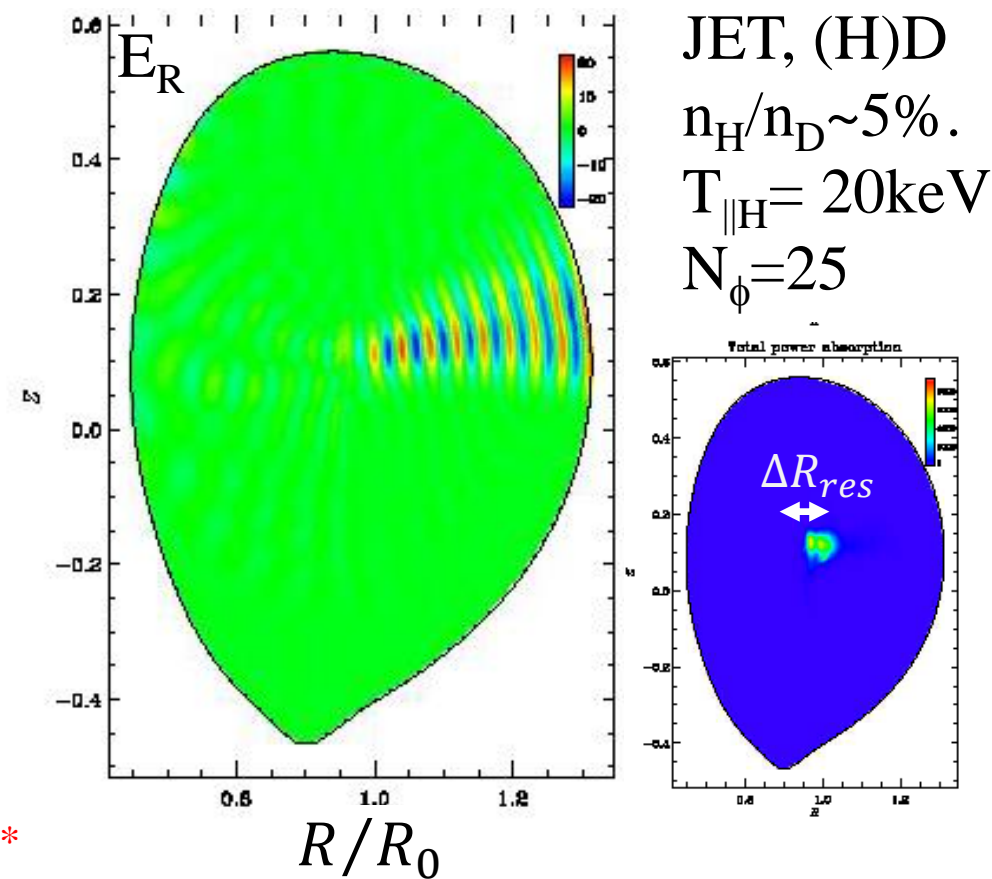


¹T.H Stix Nuclear Fusion **15**, 737 (1975)

ICRF Wave Field strong vs weak damping



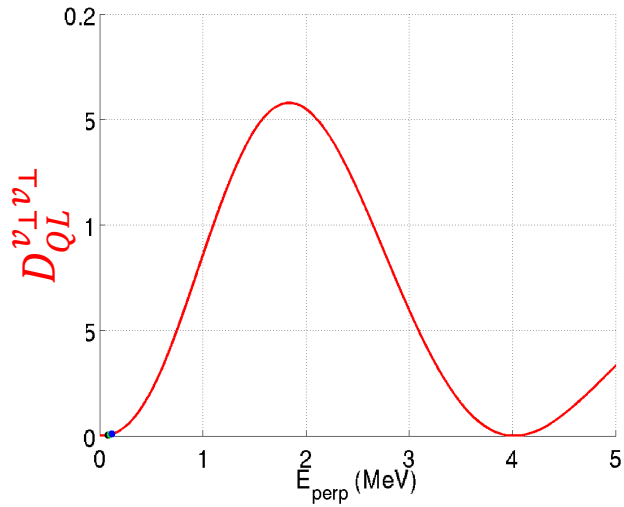
Weak damping, the wave field fills much of the cavity



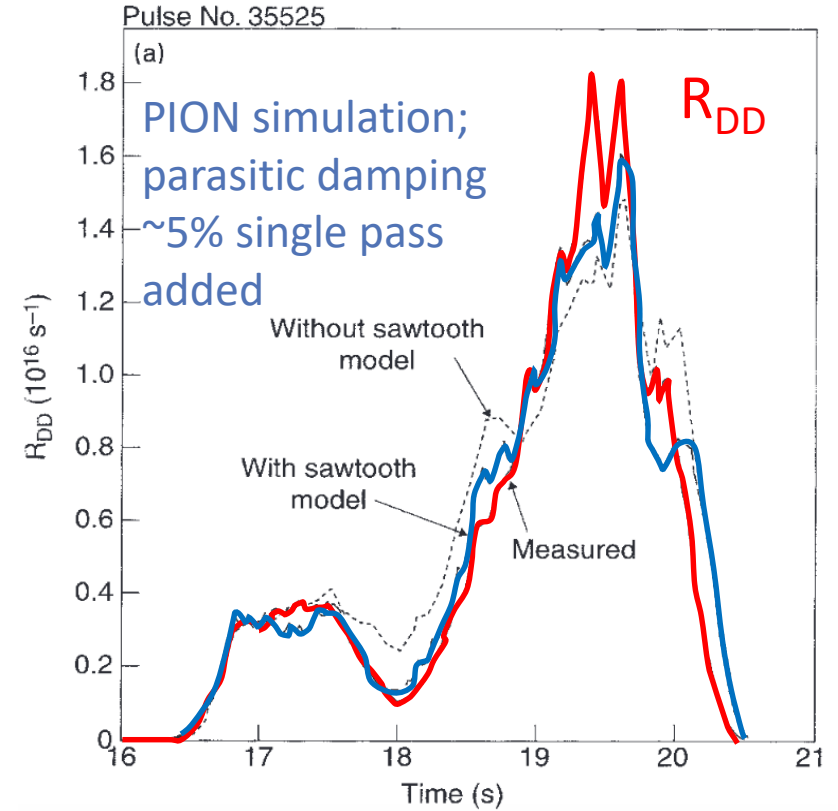
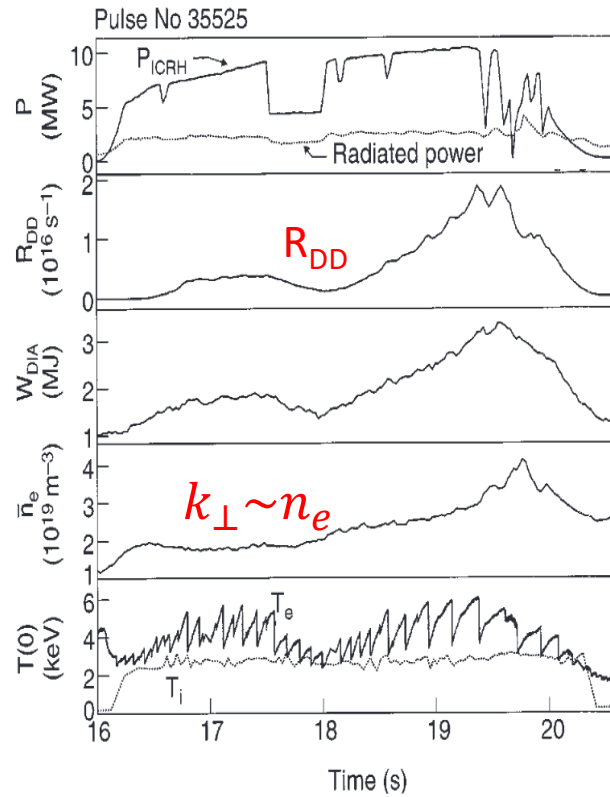
Strong damping, focussing of the wave at first passage.

*L. Villard et al., Computer Physics Reports **4**, 95 (1986). 26/06/2023

JET Experiment aimed at Fast Wave Current Drive in D plasma $\omega = 3\omega_{cD}$ near centre; Full wave code \rightarrow very weak thermal D damping.



Tail on D distribution \rightarrow damping increased due to FLR effects \rightarrow stronger tail and so on.

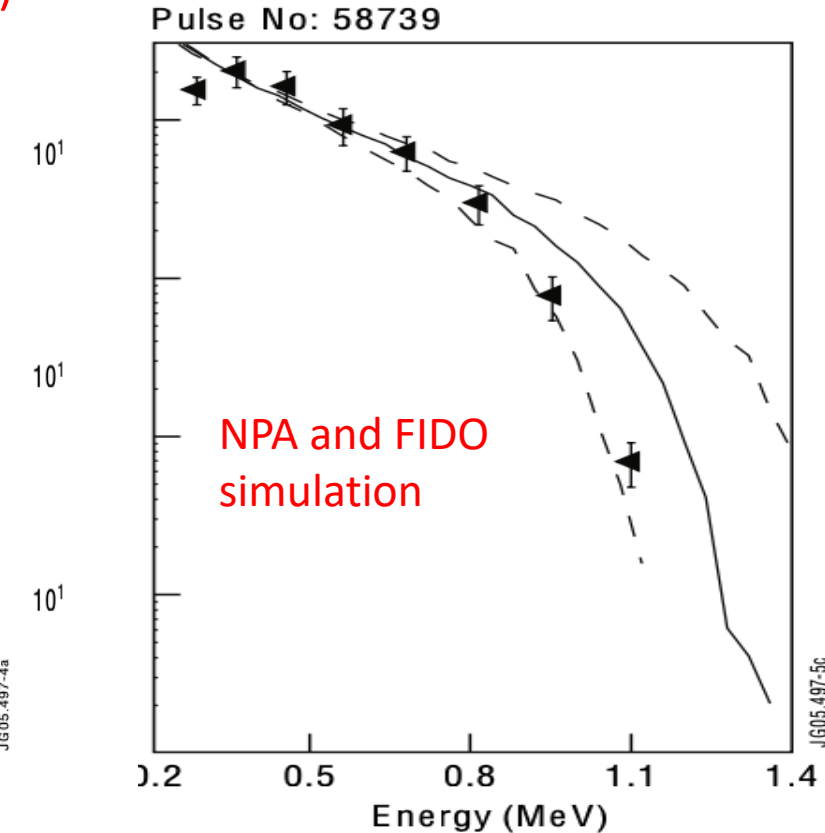
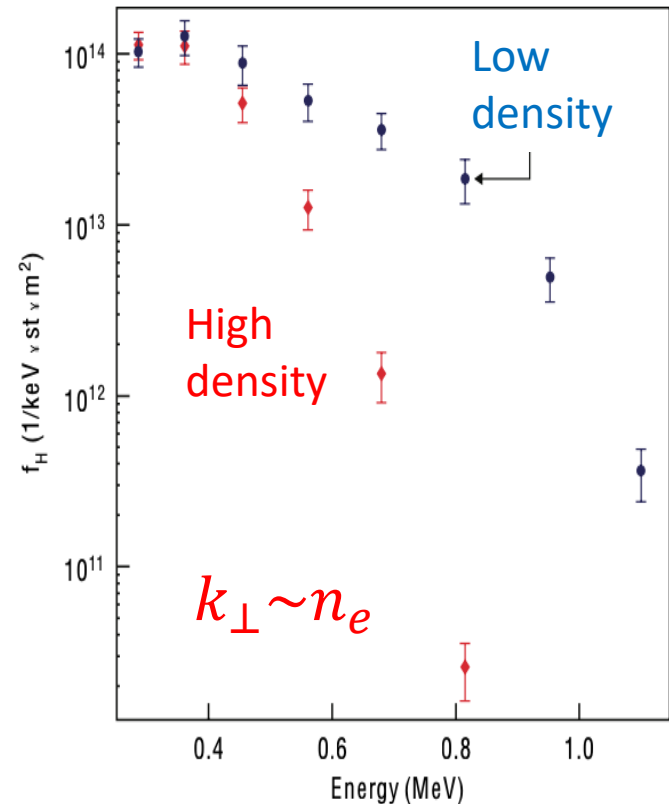
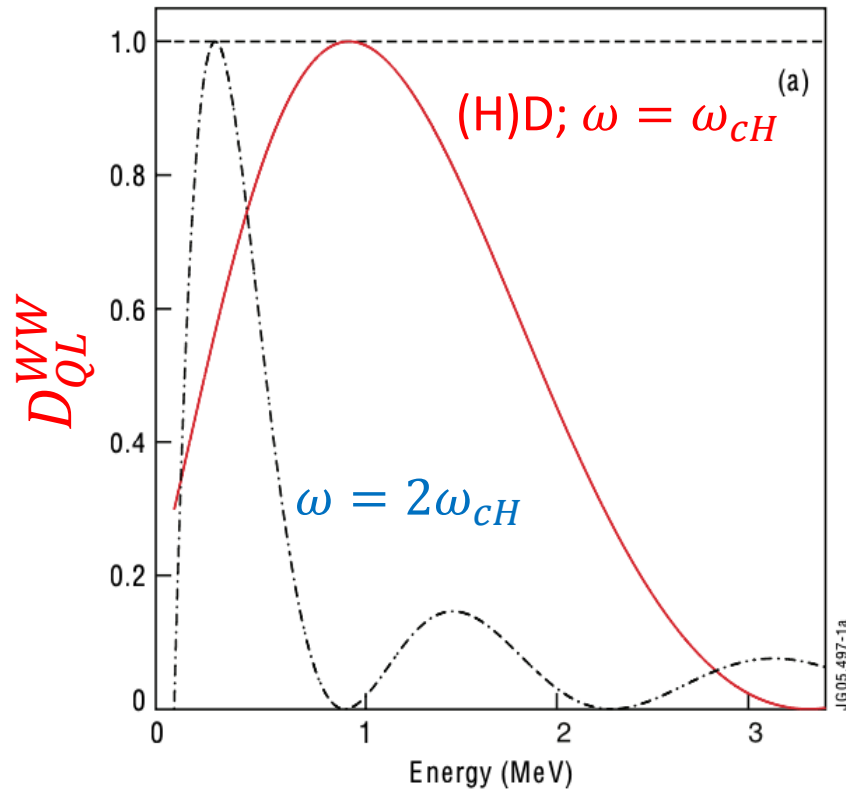


Wave propagation and F-P must be coupled self-consistently.

FLR effect; suppressed velocity space diffusion

- Second harmonic H heating in JET; $P_{ICRF} \approx 4$ MW

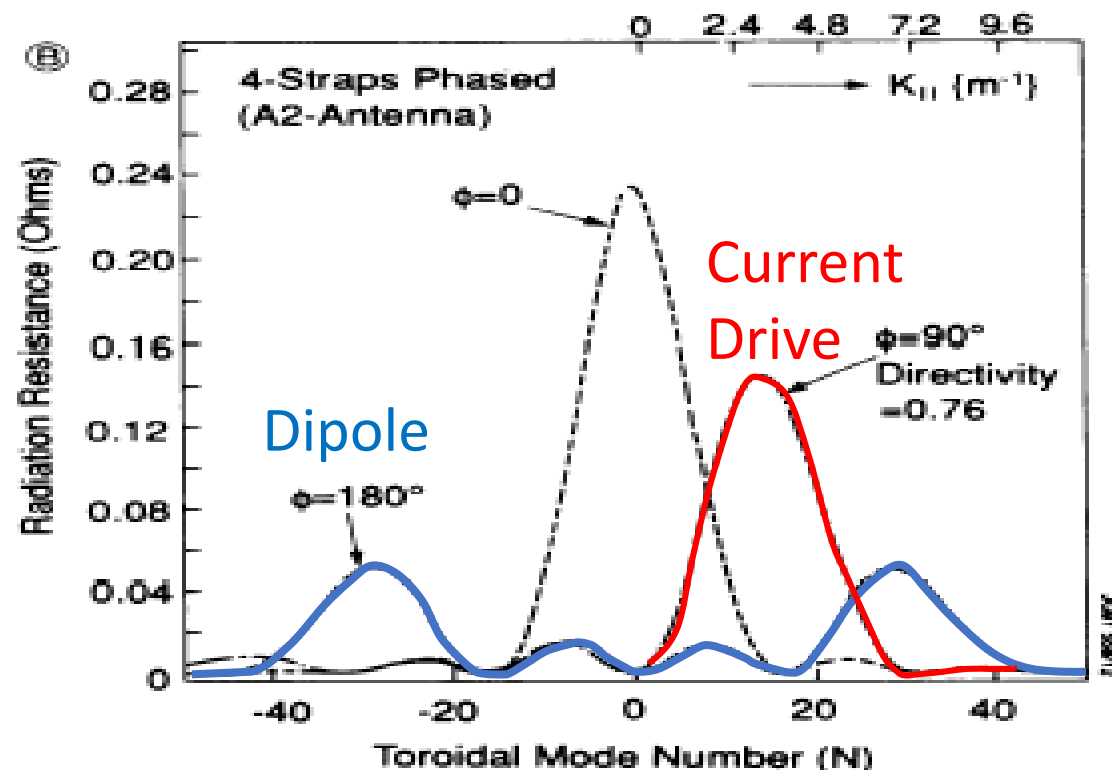
NPA measurement of $f_H(E)$



A. Salmi et al., Plasma Phys. Control. Fusion **48** (2006) 717

Influence of asymmetric ICRF antenna spectra

JET

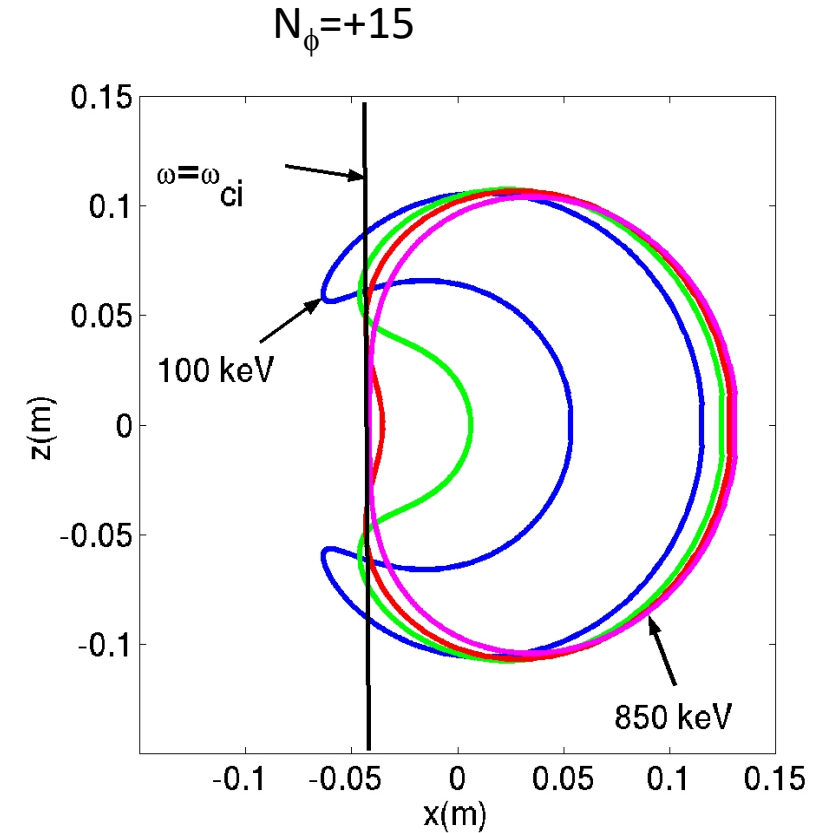
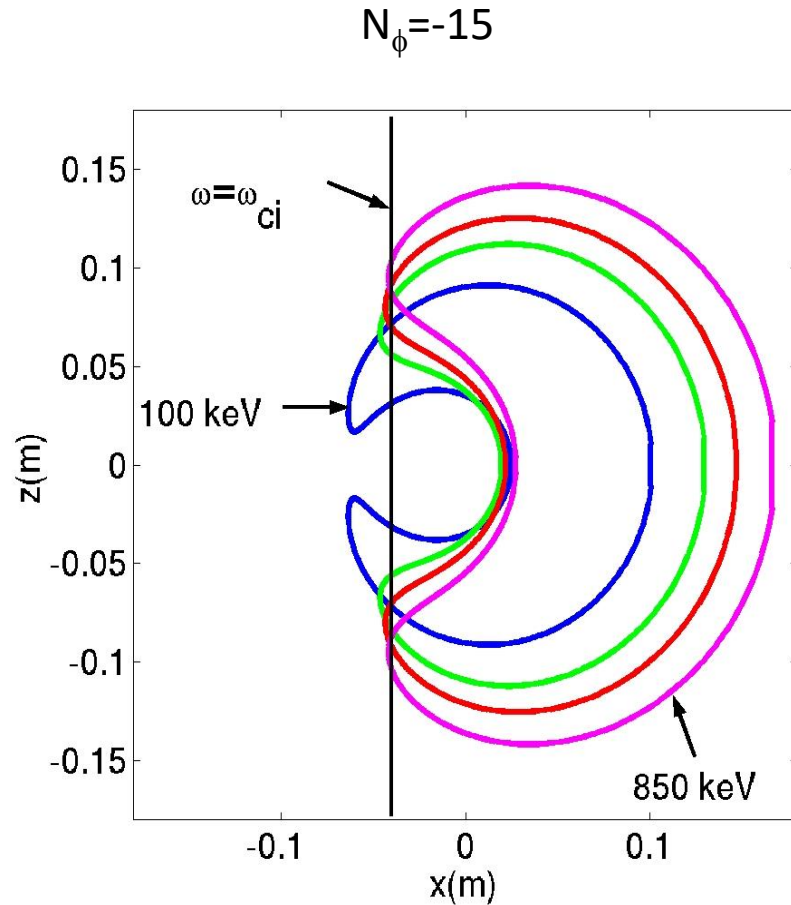


$$\langle N_\phi \rangle = \frac{\sum_{N_\phi} N_\phi P(N_\phi)}{\sum_{N_\phi} P(N_\phi)}$$

$$\frac{dP_\phi}{dt} = \frac{\langle N_\phi \rangle dW}{\omega dt}$$

JET +90° phasing → $\langle N_\phi \rangle \sim 12$; co-current propagating waves

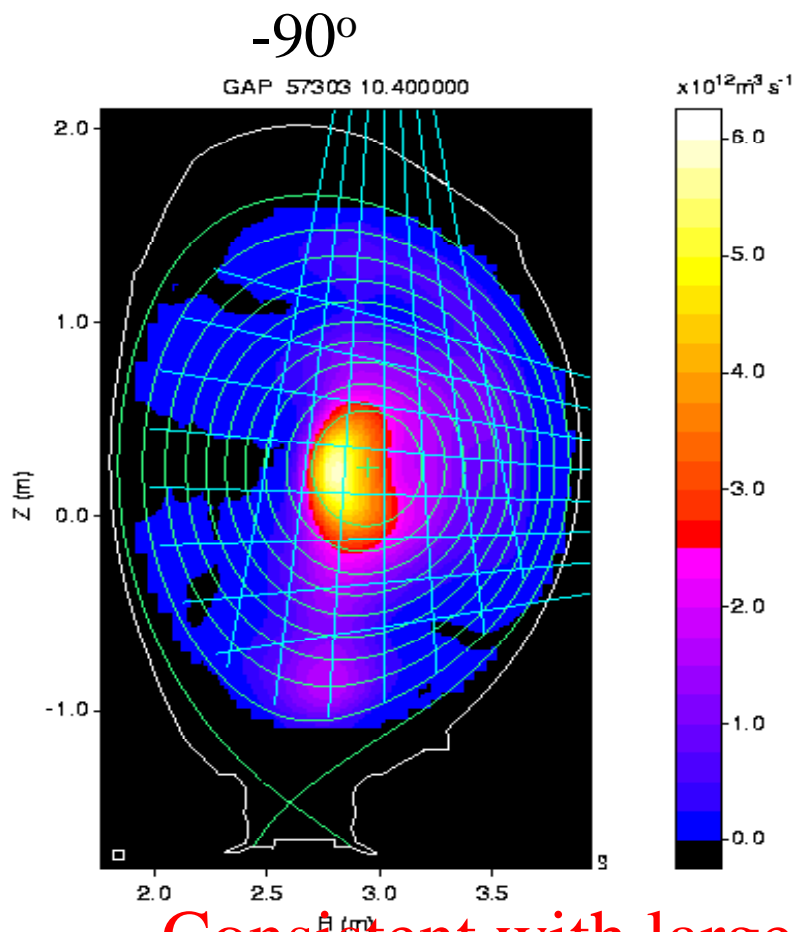
Simulation of ^3He accelerated from 100 to 850 keV



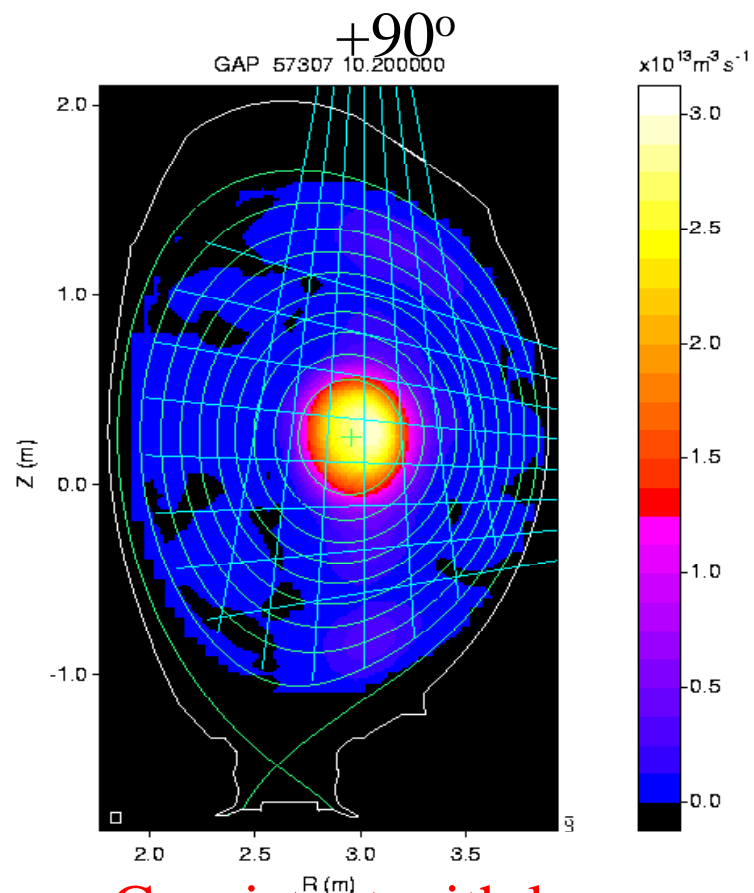
Outward movement of banana tips

Eventual de-trapping into co-passing orbit in the potato regime.

Gamma-ray measurements, interactions between fast ^3He ions and C and Be impurities with $W(^3\text{He}) \gtrsim 1.3 \text{ MeV}$



Consistent with large trapped ion fraction



Consistent with large fraction of co-passing ions

$(^3\text{He})\text{D}$ heating

$P_{\text{RF}} \approx 5 \text{ MW}$

$I_p = 2.8 \text{ MA}$

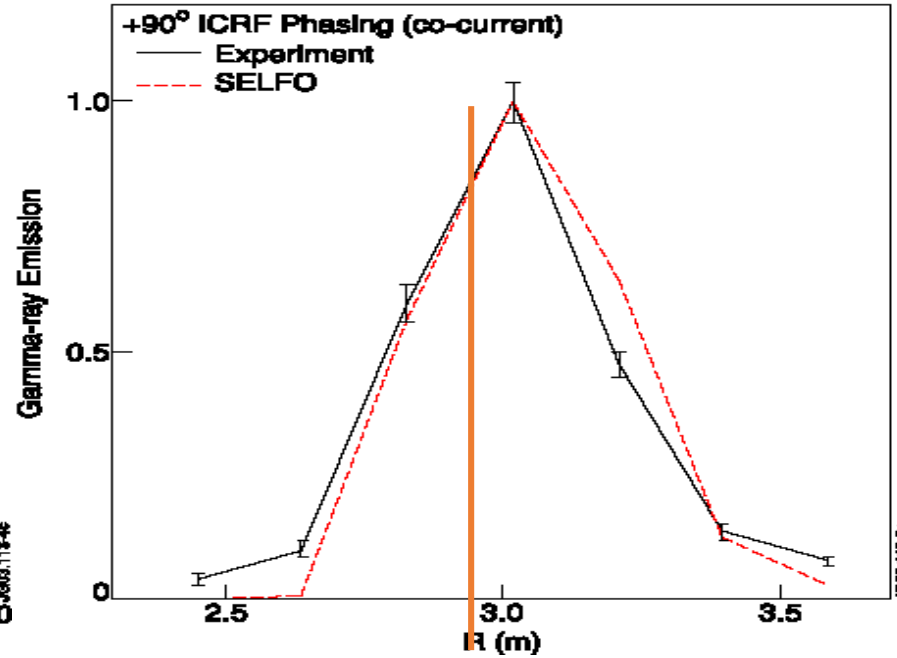
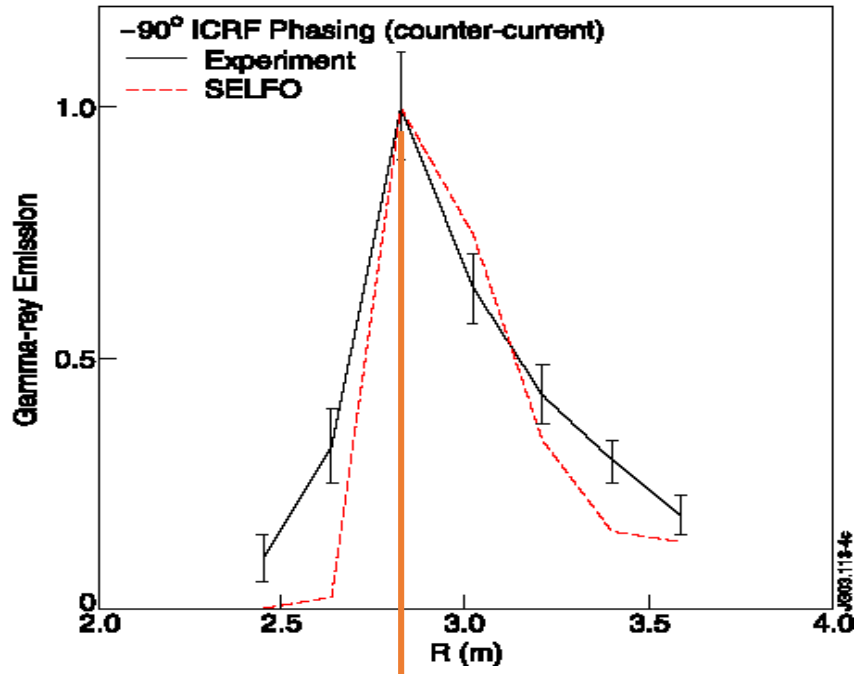
$B_T = 3.4 \text{ T}$

$f_{\text{ICRF}} = 37 \text{ MHz}$

\Rightarrow Cyclotron resonance slightly on high field side.

M. Mantsinen et al, Phys Rev Lett, 2002, **89**, 115004
L.-G. Eriksson et al, Phys Rev Lett 2004, **92**, 23500

FIDO (ICRF Monte Carlo) code simulation of the gamma-ray emission for vertical lines of sights as a function of the major radius and comparison with measurements



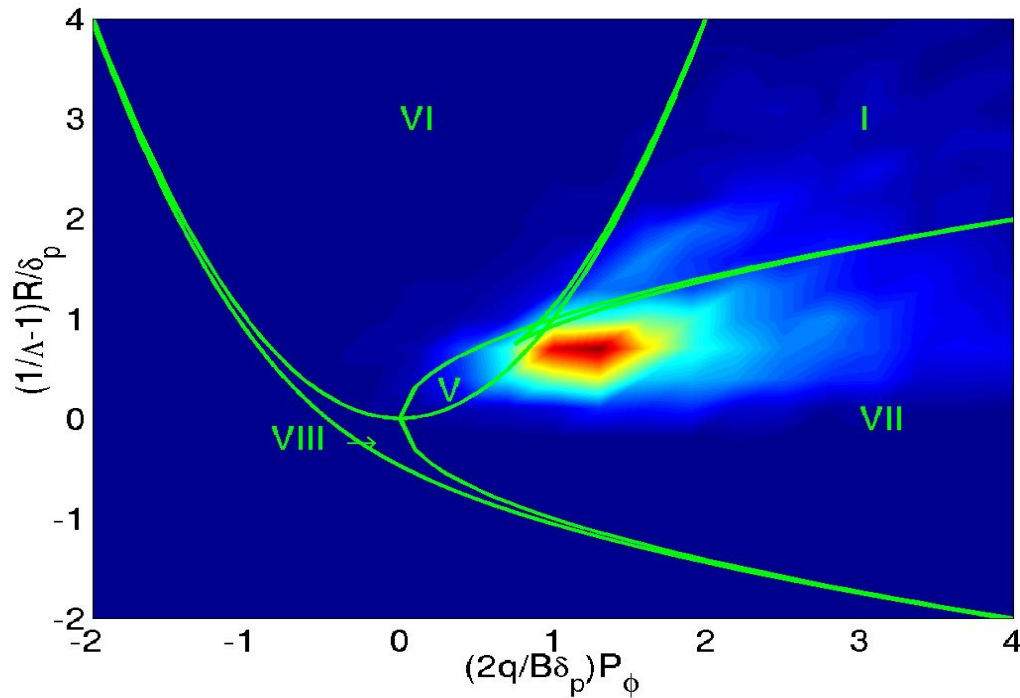
Simulation shows: trapped ions spend much time near their turning points \Rightarrow asymmetric emission around $R=3m$.

Simulation shows: the more symmetric emission around $R=3m$ is due to ions on co-passing orbits in the potato regime

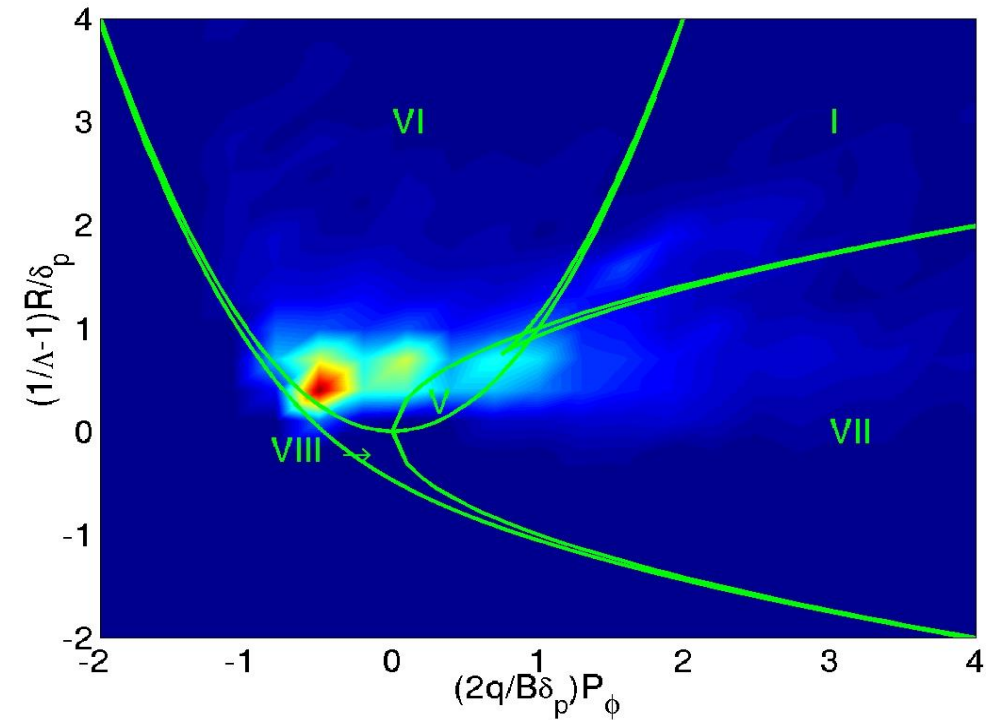
L.-G. Eriksson et al, Phys Rev Lett 2004, **92**, 23500

Simulation with Monte Carlo code FIDO solving orbit averaged Fokker Planck equation; MC particles in an orbit classification diagram; $E > 500$ keV

-90°

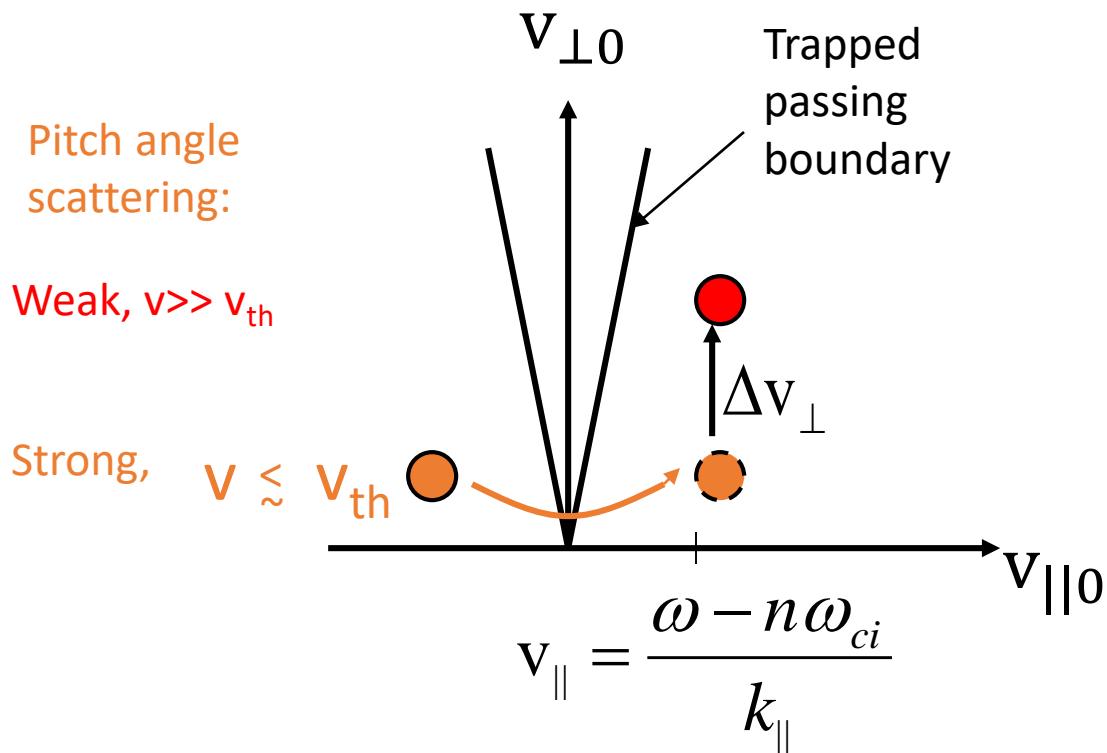


$+90^\circ$

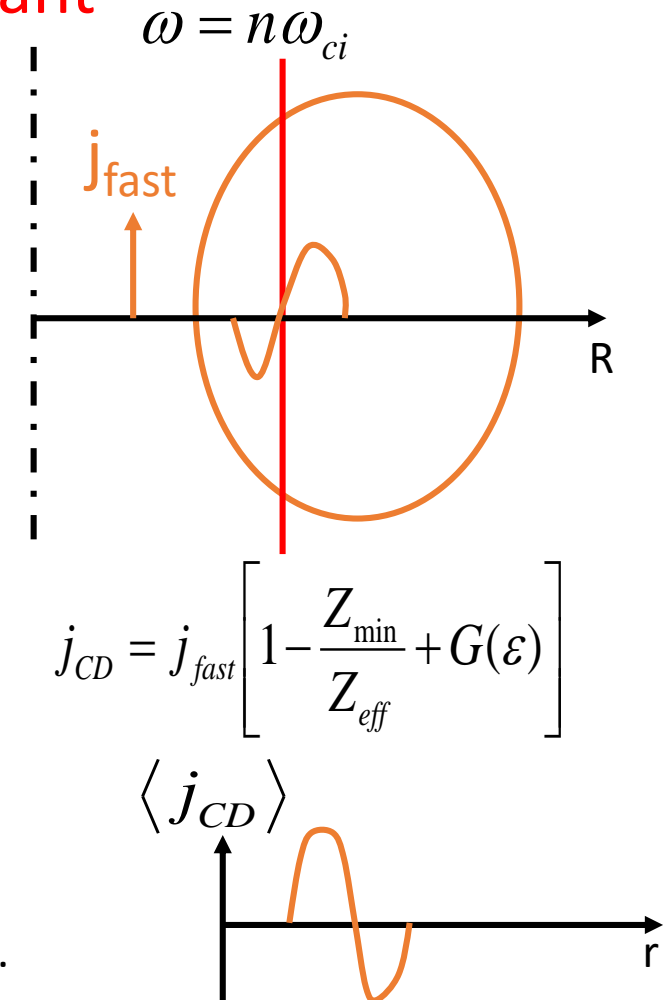


Physics of Ion Cyclotron Current Drive (ICCD)

- Fisch mechanism for ICCD using an asymmetric N_ϕ spectrum SBW limit
- As already seen, significant modification if FOW is important²



- For $(\omega - n\omega_{ci}) / k_{\parallel} > 0$ (< 0) there will be an excess particles with $v_{\parallel} > 0$ ($v_{\parallel} < 0$), i.e. a driven current.
- The effect is diminished by fast particles entering the trapped region.



¹N.J. Fisch, Rev. Mod. Phys. **59**, 175 (1987);
26/06/2023

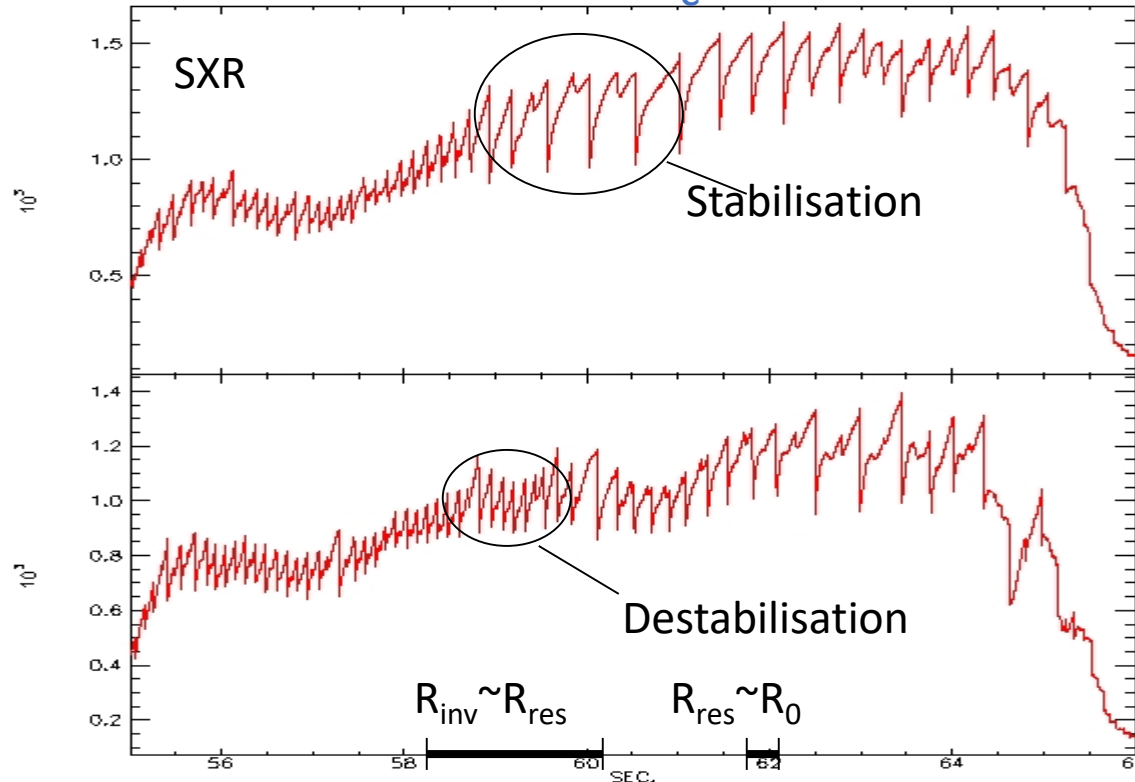
²T. Hellsten et al., Phys Rev Lett **74**, 3612 (1995).
12th ITER International School June 2023

Sawtooth behaviour with different antenna phasings on JET

- The ICCD current's dipole character should alter the shear $s = \frac{r}{q} \frac{dq}{dr}$
- Altering s near the inversion radius should affect the sawtooth frequency¹
- Orbit topology effects may also play a role².

JET H minority ICCD; $R(\omega=\omega_c)$ ramped from HFS to LFS³

$n_H/n_D \sim 30\%$



+90°, co-current
propagating
waves

-90°, counter-
current
propagating
waves

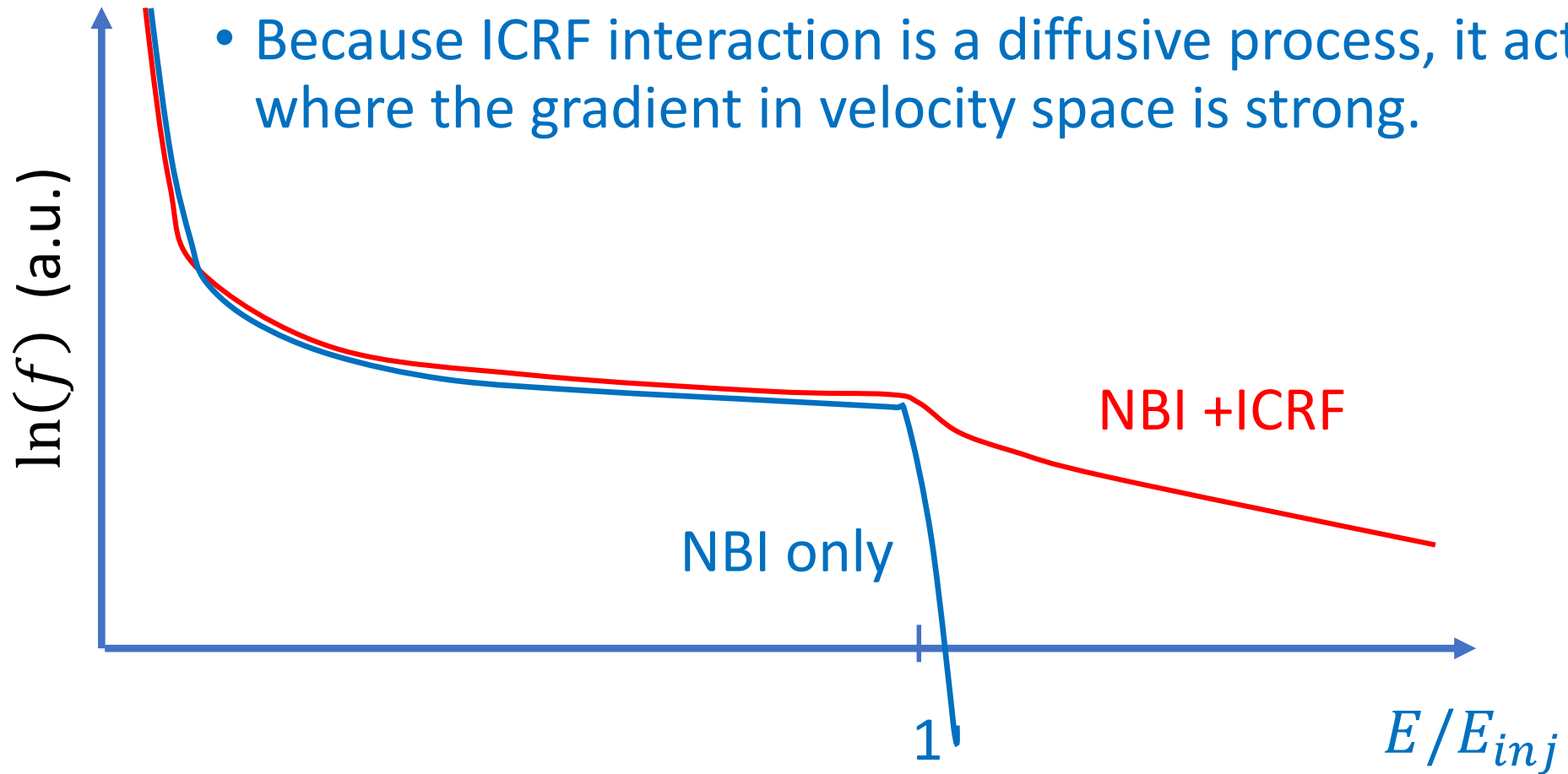
¹F. Porcelli et al., PPCF **38**, 2163 (1996);

²J.P Graves Phys Rev Lett, 2009, **102**, 065005

³L.-G. Eriksson et al. NF **46**, 951 (2006).

Combined NBI and ICRF heating cartoon

- Because ICRF interaction is a diffusive process, it acts chiefly where the gradient in velocity space is strong.



Non linear phase memory loss and random walk

- The non linear phase change between two passages of a resonance:

$$\Delta\phi = \int_{t_{res}}^{t_{res}+\tau_b} (\omega - k_{||}v_{||} - n\omega_{c,i}) d\tau_b, \quad \text{Single } N_\phi \text{ mode:} \quad \text{Chirikov criterion:} \quad \gamma = \frac{\partial\Delta\phi}{\partial v_\perp} \Delta W \gg 1 \quad \rightarrow$$

- If fulfilled, ϕ is randomised \rightarrow Random walk process
- Small banana width limit passing particles for $n = 1$; $k_{||} = 0^1$, $\gamma \approx \frac{\omega q r^2 \Delta W}{2mRv_{||}^3}$
- γ decreases with $W \rightarrow$ Super adiabatic movement at some point.
- Trapped particles are easier to randomise; FOW are important².
- A full N_ϕ spectrum tend to be necessary to explain observed ICRF acc. Ions^{2,3}

¹T.H. Stix “Waves in Plasmas” AIP 1992

²P. Helander and M. Lisak Phys. Fluids **B 4** (7), July 1992, 1927

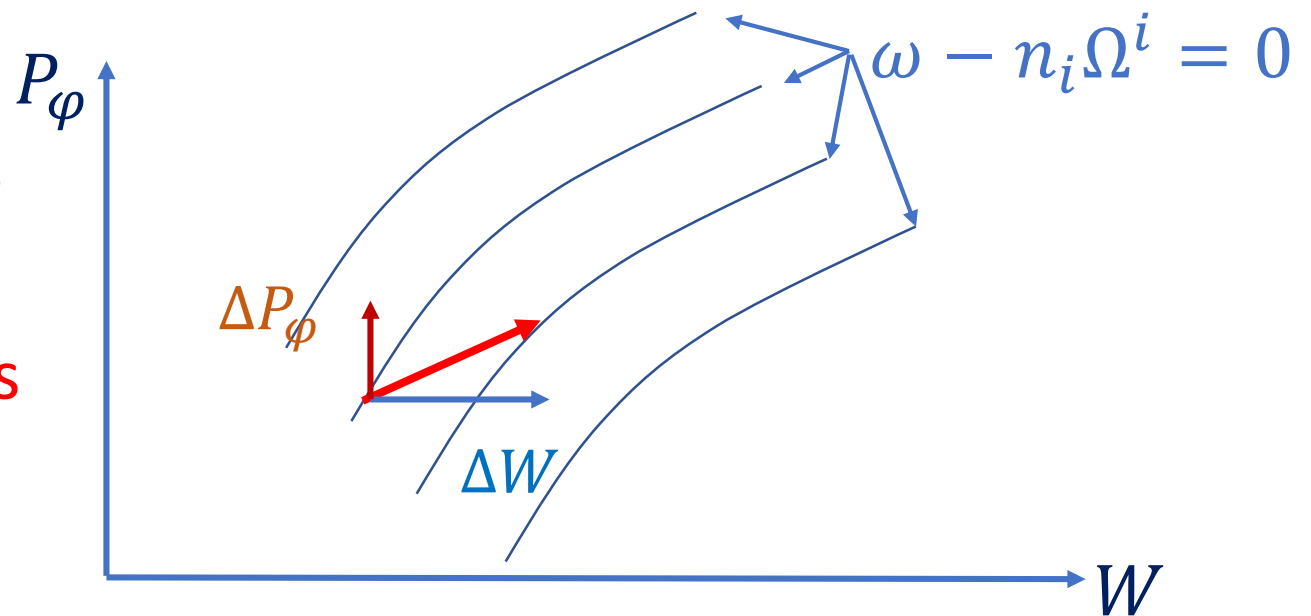
³V. Bergaud et al. Phys. Plasmas, **8**, 2001, 139

- A stricter derivation of $\langle Q(f_0) \rangle$ using quasi-linear theory^{1,2} yields,

$$D_{QL}^{WW}(n_1 = n, n_3 = N) = \omega_b \sum_{n_2} \frac{\langle (\Delta W)^2 \rangle |_{\phi}}{2\tau_b} \delta(\omega - n_i \Omega^i)$$

$$\Omega^1 = \langle \omega_c \rangle; \quad \Omega^2 = 2\pi/\tau_b; \quad \Omega^3 = \langle \dot{\phi} \rangle; \quad \langle \dots \rangle = \text{Orbit average}$$

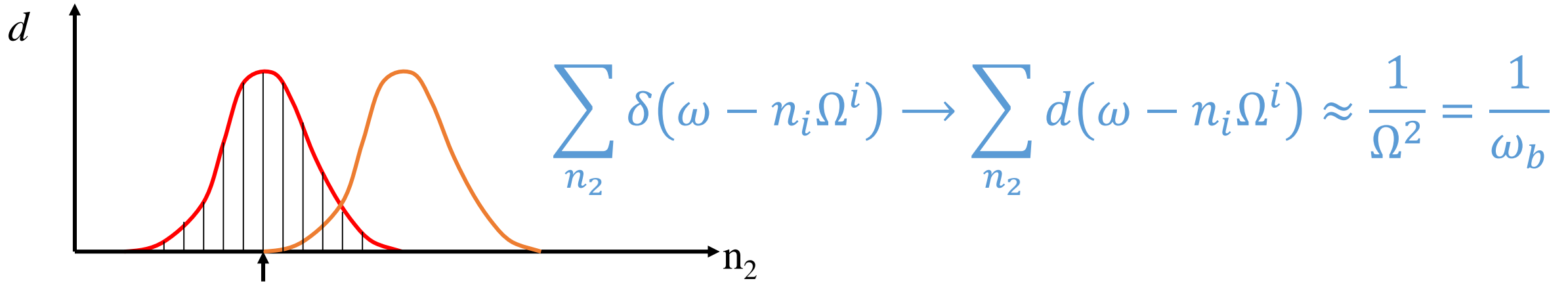
- the resonance condition $\omega - n_i \Omega^i = 0$ picks out particles that “feel” the same wave phase after one unperturbed poloidal orbit (not to be confused with $\omega - \vec{k} \cdot \vec{v}_g = n\omega_{ci}$)
- This global resonance condition, maps out surfaces in \vec{I} space.
- Strong perturbation \rightarrow kicks bridges resonant surfaces \rightarrow random walk



¹A.N. Kaufman Phys. Fluids. 1972;

²L-G. Eriksson and P. Helander, Physics of Plasmas **1**, 308, 1994

- In reality the delta functions are broadened by collisions and non-linear effects, as discussed, i.e. $\delta(\omega - n_i \Omega^i) \rightarrow d(\omega - n_i \Omega^i)$
- If the overlap is sufficient one can make the substitution* :



$$\omega - n^1 \Omega_1 - n_{res}^2 \Omega_2 - n^3 \Omega_3 = 0$$

- Thus, if we have “sufficient” overlap, i.e., $\gamma \gg 1$ we recover :

$$D_{QL}^{WW} = \sum_{res} \frac{\langle (\Delta W)^2 \rangle |_{\phi}}{2\tau_b}$$

*Becoulet et al. Phys. of Fluids **B3** (1991),

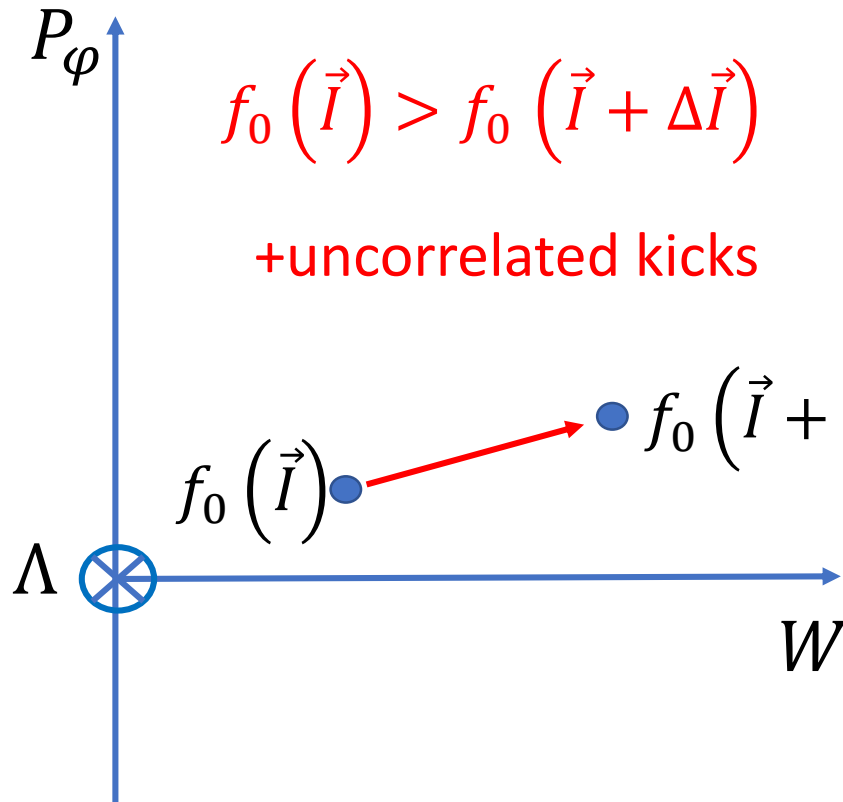
Cartoon heating vs instability

Heating

$$\Delta \vec{I} = (\Delta W, \Delta \Lambda, \Delta P_\varphi)$$

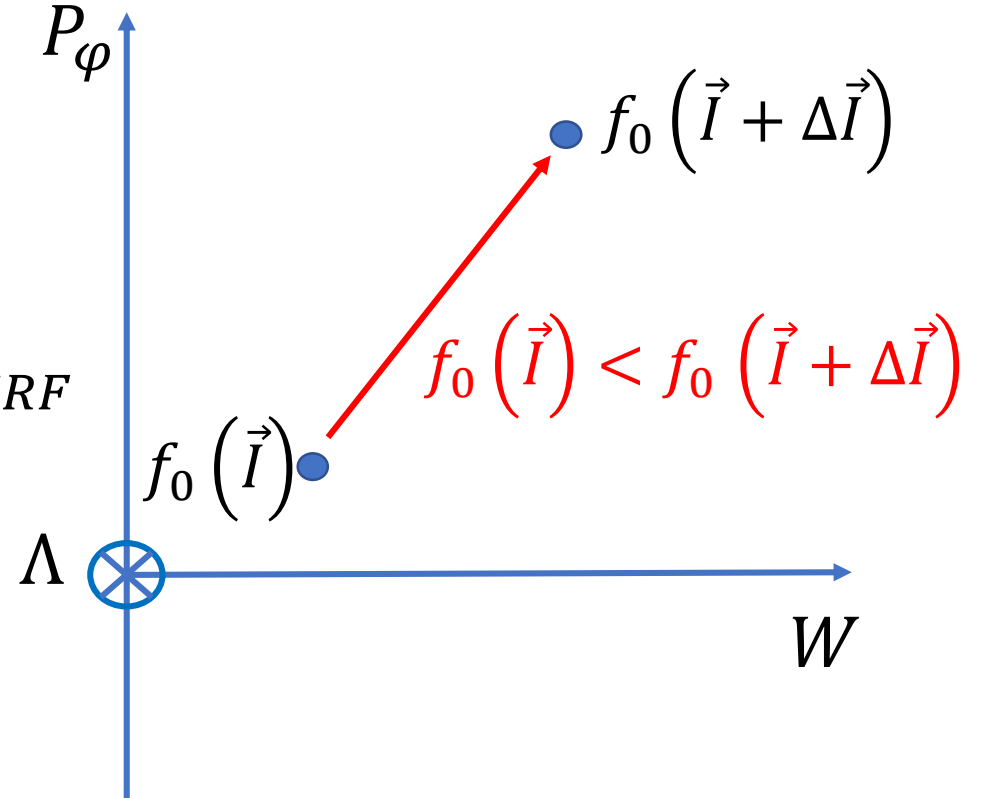
$$\Delta P_\varphi = \frac{N}{\omega} \Delta W$$

Potential for instability



Note:

$$\omega_{TAE} \ll \omega_{ICRF}$$



Round off

- Fast ion physics is an area with many facets
- In this lecture it was only possible to touch on selected subjects of fast ions sources somewhat superficially
- However, questions like to which extent a different bulk plasma species are heated by energetic ions, the currents they fast ions can drive etc. is useful general knowledge when e.g. assessing plasma scenarios.
- Finally, energetic ions are important but so are energetic electrons, not least runaway electrons and I encourage you to follow the lectures on Friday attentively.

Thank you for your attention!

Is the kick only in the perp. Direction?

- The change in the parallel energy is given by,

In general $\neq 0$

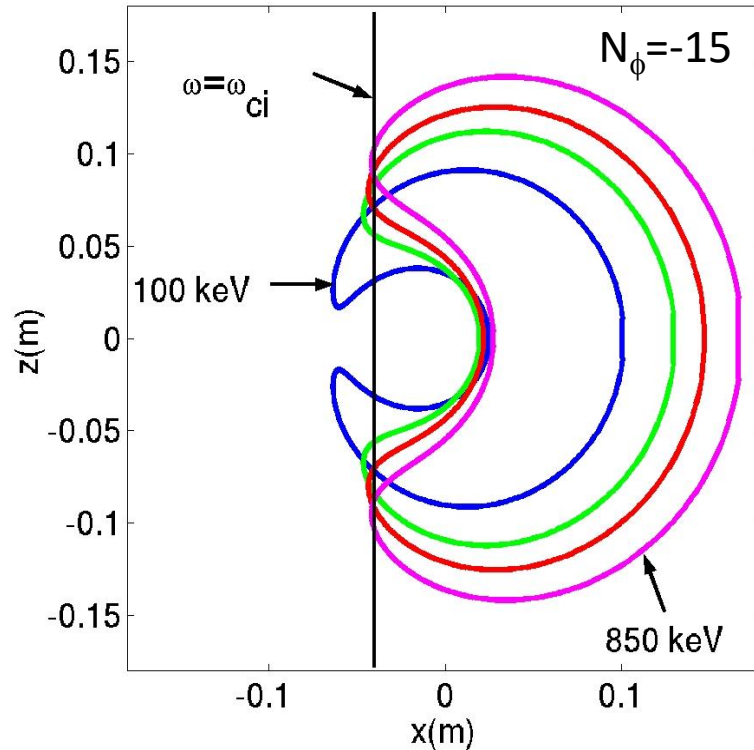
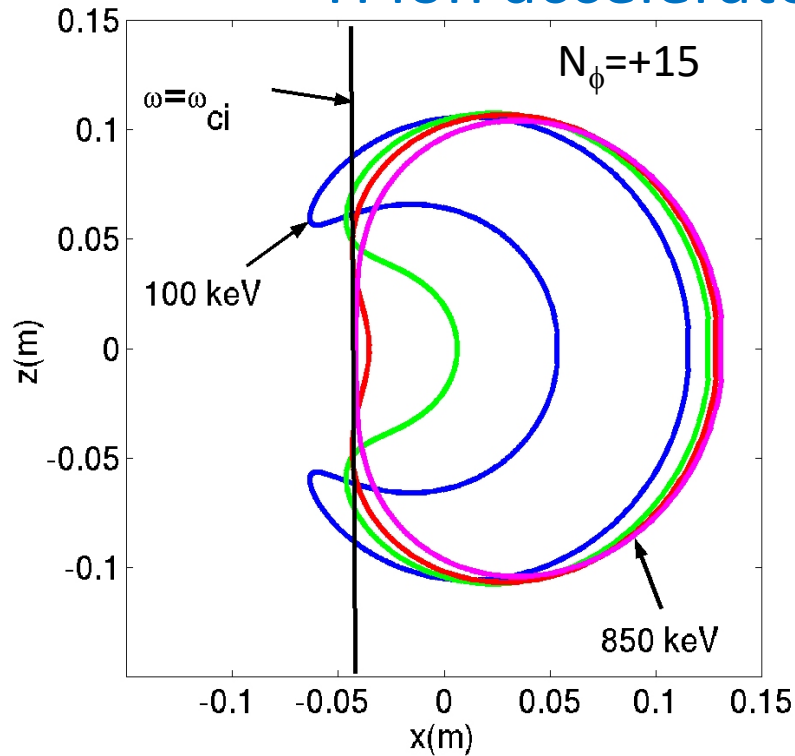
$$\Delta \frac{mv_{\parallel}^2}{2} = \int_{t_0 - \Delta t}^{t_0 + \Delta t} Ze \left(\vec{E} + \vec{v} \times \vec{B}_1 \right) \cdot \vec{v}_{\parallel} dt$$

$$= \int_{t_0 - \Delta t}^{t_0 + \Delta t} Ze \left[E_{\parallel} v_{\parallel} + \vec{B}_1 \cdot (\vec{v}_{\parallel} \times \vec{v}_{\perp}) \right] dt$$

- Thus, while the $\vec{v} \times \vec{B}_1$ term obviously is not involved in the total energy kick, it can redistribute part of it to the parallel direction.
- We can save a lot of tedious calculations by adopting a quantum mechanical perspective.

The Physics ICCD is more complicated

H ion accelerated from 100 to 850 keV



- Finite orbit width effect of trapped ions \rightarrow dipole effect on driven current (co-current further out)
- Change of orbit topology, i.e. ions driven into non-standard passing orbits

Eventual de-trapping into co-passing orbit in the potato regime \Rightarrow passing current not described by "Fisch like" theory*. Increased pressure in the centre.

Outward movement of turning points, decrease of central fast ion pressure.

*T. Hellsten, et al. Phys. Rev. Lett. **74**, 3612 (1995).

Physical intuition of P_φ

- The equation of motion in the toroidal direction reads,

$$m \frac{d(Rv_\varphi)}{dt} = Ze \left(\vec{v} \times \vec{B} \right)_\varphi = -ZeRv_r B_\theta$$

- The poloidal flux function is given by,

$$\psi = \int_0^r RB_\theta dr' \quad \longrightarrow \quad \frac{d\psi}{dt} = \vec{v}_r \cdot \nabla\psi = RB_\theta v_r \quad \longrightarrow$$

$$\frac{d}{dt} (mRv_\varphi + Ze\psi) = \frac{dP_\varphi}{dt} = 0$$

- One can see $Ze\psi$ as a “potential angular momentum”
- For trapped particles one should note that: $P_\varphi = Ze\psi_{t.p.}$ where “t.p.” stands for turning point.

Small Larmor radius

- Define orbit average:

$$\langle \dots \rangle = (2\pi)^{-3} \iiint_0^{2\pi} (\dots) d\theta^1 d\theta^2 d\theta^3 \approx \frac{1}{2\pi} \int_0^{2\pi} (\dots) d\theta^2 = \frac{1}{\tau_b} \int_0^{\tau_b} (\dots) d\tau$$

- and the orbit average of the Fokker-Planck equation yields

$$\frac{\partial f_0}{\partial t} = \langle C(f_0) \rangle + \langle S \rangle - \langle L \rangle + \langle Q(f_0) \rangle$$

- Note $\langle f \rangle = f_0(\vec{I})$, with $\vec{I} = \vec{I}(\vec{J})$

- The collision operator is conservative $\rightarrow C(f) = \frac{\partial \bar{\Gamma}_c^i}{\partial z^i}$

- With use our favourite variables, $\vec{I} = \left(v, \Lambda = \frac{\mu B_0}{E}, P_\varphi \right)$

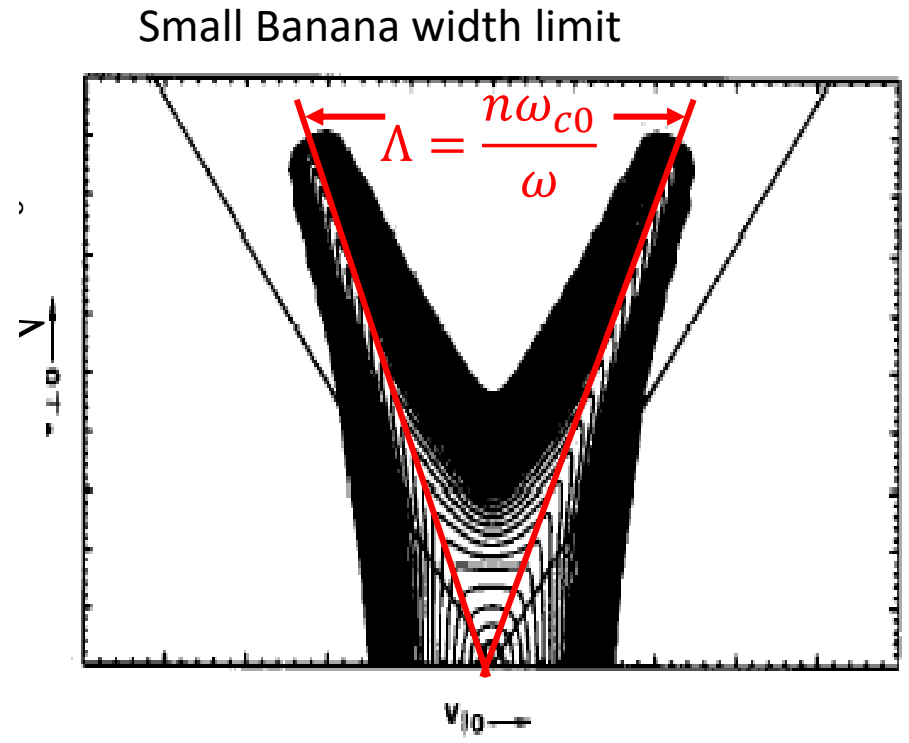
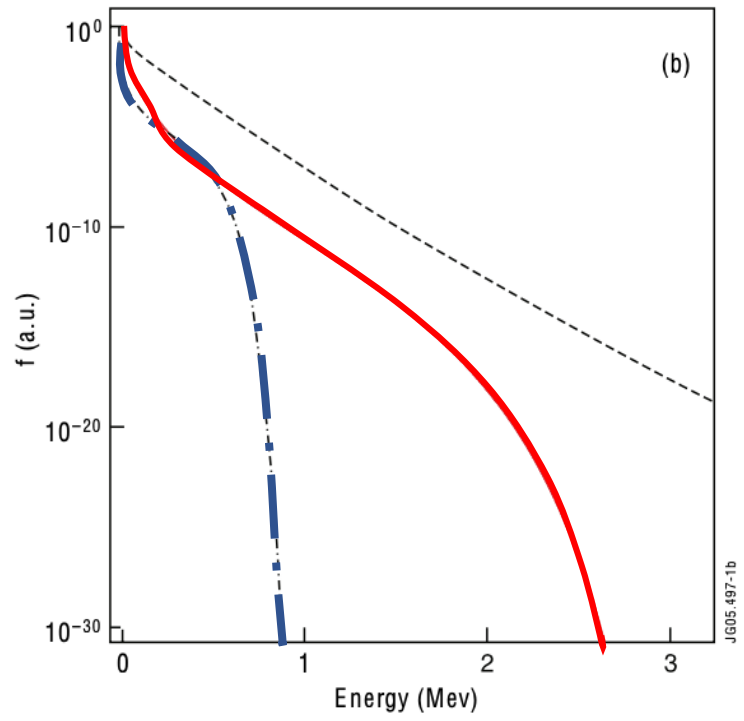
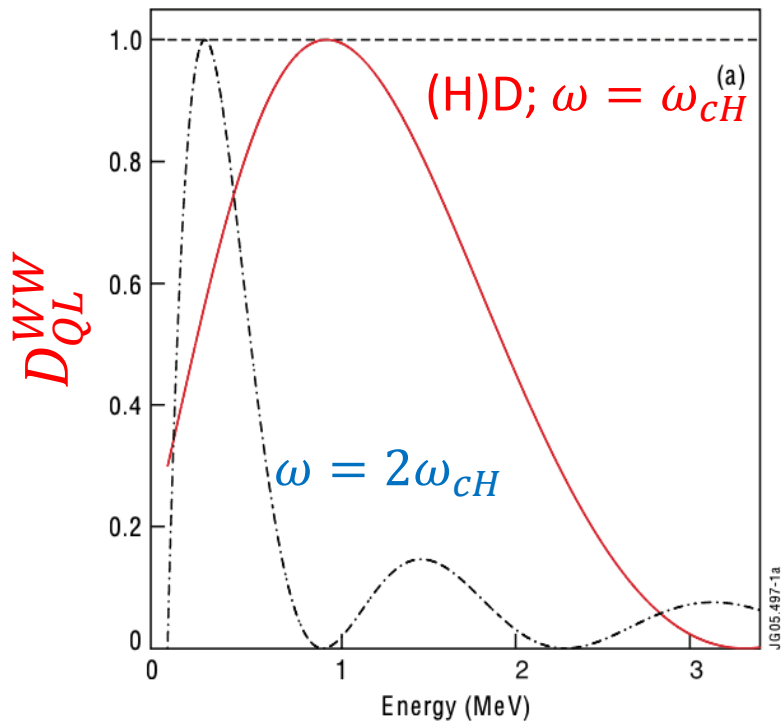
$$\langle C(f) \rangle = \frac{1}{\sqrt{g}} \frac{\partial}{\partial I^i} \left[\sqrt{g} \left\langle \bar{\Gamma}_c^j \frac{\partial I^i}{\partial z^j} \right\rangle \right]$$

$$\sqrt{g} = \left| \frac{\partial \vec{z}}{\partial (\vec{I}, \vec{\theta})} \right| = \frac{v^3 \tau_b}{4\pi m \omega_{c0}}$$

Properties of ICRF heated distributions

- The collisions are much stronger at low energies than at high \rightarrow development of a non-Maxwellian tail on the distribution.

- $\Delta\Lambda = \frac{n\omega_{c0} - \Lambda\omega}{\omega W} \Delta W \rightarrow \Lambda \xrightarrow{\Delta W > 0} \frac{n\omega_{c0}}{\omega}$; ion with t.p. at resonance, has $\Lambda = \frac{n\omega_{c0}}{\omega}$



G.D. Kerbel and M.G McCoy, Phys. Fluids **28** (1985) 3629.